LICENSEE EVENT REPORT

CONTROL BLOCK [ ] [ ] [ ] [ ] (1) (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

1 | C T M N S 1 (2) | 0 0 - 0 0 0 0 0 - 0 0 (3) | 4 1 1 1 1 (4) | (5)

8 9 LICENSEE CODE 14 15 LICENSE NUMBER 25 26 LICENSE TYPE 30 57 CAT 58

CON'T

0 1 | REPORT SOURCE L (6) | 0 5 0 0 0 2 4 5 (7) | 1 0 0 5 8 0 (8) | 1 0 1 6 8 0 (9)

8 60 61 DOCKET NUMBER 68 69 EVENT DATE 74 75 REPORT DATE 80

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

0 2 | On October 5, 1980 at 1530 hours, the Local Leak Rate Surveillance Testing Program

0 3 | identified two (2) Main Steam Isolation Valves that could not meet the required leak

0 4 | rate (T.S. 4.7.f.2.C). The plant is presently shutdown for refueling. There were no

0 5 | consequences.

0 6 | Similar occurrences: RO-78-6/1P, RO-79-19/3L

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SYSTEM CODE | CAUSE CODE | CAUSE SUBCODE | COMPONENT CODE | COMP. SUBCODE | VALVE SUBCODE

0 9 | C D (11) | E (12) | B (13) | V A L V E X (14) | F (15) | D (16)

8 9 10 11 12 13 18 19 20

(17) LER/RO REPORT NUMBER | EVENT YEAR | SEQUENTIAL REPORT NO. | OCCURRENCE CODE | REPORT TYPE | REVISION NO.

8 0 | — | 0 1 4 | / | 0 1 | T | — | 0

21 22 23 24 26 27 28 29 30 31 32

ACTION TAKEN | FUTURE ACTION | EFFECT ON PLANT | SHUTDOWN METHOD | HOURS (22) | ATTACHMENT SUBMITTED | NPRD-4 FORM SUB. | PRIME COMP. SUPPLIER | COMPONENT MANUFACTURER

X (18) | B (19) | Z (20) | Z (21) | 0 0 0 0 | Y (23) | Y (24) | N (25) | C 6 6 5 (26)

33 34 35 36 37 40 41 42 43 44 47

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

1 0 | An investigation into the cause of failure is in progress. Additional information

1 1 | concerning these valves and other valves failing local leak rate testing will be

1 2 | included in an update report following completion of the refueling outage.

1 3 |

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FACILITY STATUS | % POWER | OTHER STATUS (30) | METHOD OF DISCOVERY | DISCOVERY DESCRIPTION (32)

1 5 | H (28) | 0 0 0 (29) | NA | B (31) | Local Leak Rate Testing

8 9 10 12 13 44 45 46 80

ACTIVITY RELEASED | CONTENT OF RELEASE | AMOUNT OF ACTIVITY (35) | LOCATION OF RELEASE (36)

1 6 | Z (33) | Z (34) | NA | NA

8 9 10 11 44 45 80

PERSONNEL EXPOSURES

NUMBER | TYPE | DESCRIPTION (39)

1 7 | 0 0 0 (37) | Z (38) | NA

8 9 11 12 13

PERSONNEL INJURIES

NUMBER | DESCRIPTION (41)

1 8 | 0 0 0 (40) | Z | NA

8 9 11 12 80

LOSS OF OR DAMAGE TO FACILITY (43)

TYPE | DESCRIPTION

1 9 | Z (42) | NA

8 9 10 80

PUBLICITY

ISSUED | DESCRIPTION (45) | NRC USE ONLY

2 0 | N (44) | NA

8 9 68 69 80

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NAME OF PREPARER

NRC FORM 366 (7-77)

ATTACHMENT TO LER 80-14/1T
NORTHEAST NUCLEAR ENERGY COMPANY
MILLSTONE NUCLEAR POWER STATION - UNIT 1
PROVISIONAL LICENSE NUMBER DPR-21
DOCKET NUMBER 50-245

IDENTIFICATION OF OCCURRENCE

Failure of two primary containment isolation valves to meet the Technical Specification limit for Local Leak Rate Testing.

CONDITIONS PRIOR TO OCCURRENCE

The unit was shut down on October 4, 1980 for a planned refueling outage.

DESCRIPTION OF OCCURRENCE

On October 5, 1980 at 1530 hours, after performance of Local Leak Rate Testing of the Main Steam Isolation Valves, it was discovered that 1-MS-1D and 1-MS-2C failed to meet the acceptance criteria of 11.5 SCFH maximum leakage (T.S. 4.7.f.2.C). The as-found leakage of 1-MS-1D was 15.48 SCFH; the as-found leakage of 1-MS-2C was 19.66 SCFH.

APPARENT CAUSE OF OCCURRENCE

The cause of failure of the subject valves to meet the required leak rate is not known at the present time.

ANALYSIS OF OCCURRENCE

Failure of the two Main Steam Isolation Valves in question to meet the required acceptable leakage criteria did not compromise primary containment integrity. The redundant Main Steam Isolation Valve on each of the affected steam lines were found to be within the acceptable limit and would have provided the required isolation function if needed.

CORRECTIVE ACTION

An investigation of the cause of the subject valves to fail to meet the required leakage criteria is in progress. Plant personnel will continue to test the remaining containment isolation valves.

Information regarding 1-MS-1D, 1-MS-2C and other valves failing local leak rate tests, including corrective action and retests, will be provided in an update report upon completion of the refueling outage.

Similar occurrences: RO-78-6/1P, RO-79-19/3L