Docket Nos. 50-317 and 50-318

> Mr. G. C. Creel Vice President Nuclear Energy Baltimore Gas & Electric Company Calvert Cliffs Nuclear Power Plant MD Rtes 2 & 4 P. O. Box 1535 Lusby, Maryland 20667

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Dear Mr. Creel:

SUBJECT: PROPOSED TECHNICAL SPECIFICATION AMENDMENT RELATING TO HIGH PRESSURE SAFETY INJECTION PUMP OPERABILITY IN MODE 3 - REQUEST FOR ADDITIONAL INFORMATION

Re: Calvert Cliffs, Unit 1 (Tac No. 75562) and Unit 2 (Tac No. 75563)

By letter dated December 20, 1989, you proposed the subject changes to the Calvert Cliffs, Units 1 and 2, Technical Specifications. The staff is currently reviewing your request and has identified the need for additional information. The requested information has been discussed with your staff and is provided in the enclosure to this letter.

We request a prompt response to support our review schedule.

The reporting and/or recordkeeping requirements contained in this letter affect fewer than ten respondents, therefore, OMB clearance is not required under P. L. 96-511.

Sincerely,

Original signed by

Daniel G. McDonald, Senior Project Manager Project Directorate I-1 Division of Reactor Projects I/II Office of Nuclear Reactor Regulation

cc: See next page			
Enclosure: As stated			
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Mr. G. C. Creel Baltimore Gas & Electric Company

cc:

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Ms. Patricia Birnie Co-Director Maryland Safe Energy Coalition P. O. Rox 902 Columbia, Maryland 21044

ENCLOSURE

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REQUEST FOR ADDITIONAL INFORMATION CONCERNING TECHNICAL SPECIFICATION CHANGE ON HIGH PRESSURE SAFETY INJECTION PUMP OPERABILITY IN MODE 3 FOR CALVERT CLIFFS UNIT 1 AND 2 DOCKET NO. 50-317

- In your LOCA evaluation during Mode 3 with reactor coolant temperature below 350°F, you have assumed that the fission product decay heat is based on 110 percent of 1971 ANS proposed standard. It is the staff's position that the fission product decay heat following a LOCA shall be based on 120 percent of 1971 ANS proposed standard per the requirements of 10 CFR 50, Appendix K. Provide the results of your evaluation using this corrected assumption.
- Provide a discussion of the consequences of a SBLOCA during Mode 3 with reactor temperature below 350°F and the HPSI pumps placed in PULL-TO-LOCK. Include the following:
 - a. The time available for operator action to initiate HPSI pumps following identification of an SBLOCA. Describe the actions needed to manually initiate HPSI pumps.
 - b. Describe the procedures available to operators for the above postulated SBLOCA. Discuss the time assumed between the initiation of the event and positive event identification.
- Discuss the consequences of manual startup of HPSI pumps during Mode 3 relative to LTOP concerns.