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January 13, 1990 C311-90-2003

U. S. Nuclear Regulatory Commission Attn: Document Control Desk Washington, D.C. 20555

Dear Sir:

Three Mile Island Nuclear Station, Unit I (TMI-1)
Operating License No. DPR-50
Docket No. 50-289
Monthly Operating Report
December 1989

Enclosed are two copies of the December 1989 Monthly Operating Report for Three Mile Island Nuclear Station, Unit 1.

Sincerely.

Vice President & Director, TMI-1

HDH/WGH:spb

cc: W. Russell, USNRC

F. Young, USNRC

Attachments

9001230023 891231 FDR ADOCK 05000289

1624

OPERATIONS SUMMARY DECEMBER 1989

It was previously reported that the plant tripped on November 29 due to a fault in the turbine electrohydraulic control system. The unit was returned to service and on the grid at approximately 6:00 a.m. on December 1, 1989. The plant was operated at 100% power until December 15, when useful core life at 100% power was reached and a slow power reduction to coastdown for the 8R refueling cutage was initiated.

MAJOR SAFETY RELATED MAINTENANCE

During December, the following major maintenance activities were per ormed on safety-related equipment:

Emergency Diesel Generator EG-Y-1B

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Emergency Diesel Generator EG-Y-1B was removed from service in December to perform the scheduled annual inspection and to accomplish modifications. All inspections were completed with no deficiencies found. The modifications completed included the installation of a high crankcase pressure trip reset circuit, the elimination of a diesel generator breaker indicating light sneak circuit, the installation of an air start solenoid valve test switch, installation of an "ES" test light for the stop push button and replacement of an up to frequency relay. Upon completion of the annual inspection and modification work, the diesel was satisfactorilly tested per the test sequence and returned to service.

Liquid Waste Disposal System Valve WDL-V-272

Valve WDL-V-272 was removed from service to replace a broken stem. A new bonnet assembly was withdrawn from warehouse stock and installed on the existing valve body. Reassembly of the valve was completed with installation of the valve chain operator. The valve was tested satisfactorilly and returned to service.

Air Intake Tunnel Fire Service Piping at FS-V-178

The Air Intake Tunnel Fire Service piping was removed from service in December to replace a split section of pipe. The split was attributed to the freezing of residual water in the piping. A weep hole was drilled into the replacement piping section to permit draining. Upon completion of the work, the Fire Service piping was declared operable and returned to service.

Auxiliary and Fuel Handling Building Exhaust Filters AH-E-2 A/B/D

The charcoal filters in the Auxiliary and Fuel Handling Building Ventilation Exhaust (AH-E-2 A/B/D) were replaced after failing a halide leak test. One-hundred and two charcoal filters were replaced in each of the filter units. AH-E-2 A and D tested satisfactorilly after the filter replacement but AH-E-2B failed. The filters in the B unit were replaced a second time and the unit retested. The retest was satisfactorilly completed and the three units were returned to service.

DOCKET NO. 50-289
DATE 12-31-89
COMPLETED BY C.W. Smyth
TELEPHONE (717) 948-8551

OPERATING STATUS

UNIT NAME: REPORTING PERIOD: LICENSED THERMAL POWER (MWT): NAMEPLATE RATING (GROSS MWE): DESIGN ELECTRICAL RATING(NET MWE): MAXIMUM DEPENDABLE CAPACITY (GROSS MWE MAXIMUM DEPENDABLE CAPACITY (NET MWE):	ND UNIT 1 ER ,1989. 2568. 871. 819.	NOTES	
IF CHANGES OCCUR IN (ITEMS 3-7) SINCE	LAST REPORT	, GIVE REASON	vs:
POWER LEVEL TO WHICH RESTRICTED, IF AN REASONS FOR RESTRICTIONS, IF ANY:	Y (NET MWE)		
	THIS MONTH	YR-TO-DATE	CUMMULATIVE
INLINE THE DECEMBER SERVER			
HOURS IN REPORTING PERIOD	744.	8760.	134401
HOURS IN REPORTING PERIOD NUMBER OF HOURS REACTOR WAS CRITICAL	744. 741.1	8760. 8717.2	134401 61998.
HOURS IN REPORTING PERIOD NUMBER OF HOURS REACTOR WAS CRITICAL REACTOR RESERVE SHUTDOWN HOURS HOURS GENERATOR ON-LINE	744. 741.1 2.9	8760. 8717.2 42.8	134401 61998. 2002.
HOURS IN REPORTING PERIOD NUMBER OF HOURS REACTOR WAS CRITICAL REACTOR RESERVE SHUTDOWN HOURS HOURS GENERATOR ON-LINE UNIT RESERVE SHUTDOWN HOURS	744. 741.1 2.9 738.0	8760. 8717.2 42.8 8714.1	134401 61998.0 2002.0 60996.
HOURS IN REPORTING PERIOD NUMBER OF HOURS REACTOR WAS CRITICAL REACTOR RESERVE SHUTDOWN HOURS HOURS GENERATOR ON-LINE JNIT RESERVE SHUTDOWN HOURS GROSS THERMAL ENERGY GENERATED (MWH)	744. 741.1 2.9 738.0 0.0 1769455.	8760. 8717.2 42.8 8714.1 0.0 22212789.	134401 61998.0 2002.0 60996.
HOURS IN REPORTING PERIOD NUMBER OF HOURS REACTOR WAS CRITICAL REACTOR RESERVE SHUTDOWN HOURS HOURS GENERATOR ON-LINE JNIT RESERVE SHUTDOWN HOURS GROSS THERMAL ENERGY GENERATED (MWH)	744. 741.1 2.9 738.0 0.0 1769455.	8760. 8717.2 42.8 8714.1 0.0 22212789. 7632284.	134401 61998.0 2002.0 60996. 0.0 149571894 50306669
GROSS ELECTRICAL ENERGY GENERATED (MWH	614222.	7632284.	50306669
GROSS ELECTRICAL ENERGY GENERATED (MWH) NET ELECTRICAL ENERGY GENERATED (MWH) UNIT SERVICE FACTOR	578261. 99.2	7632284. 7216779. 99.5	50306669 47186489 45
GROSS ELECTRICAL ENERGY GENERATED (MWH) NET ELECTRICAL ENERGY GENERATED (MWH) JNIT SERVICE FACTOR JNIT AVAILABILITY FACTOR	578261. 99.2 99.2	7632284. 7216779. 99.5 99.5	50306669 47186489 45
GROSS ELECTRICAL ENERGY GENERATED (MWH) NET ELECTRICAL ENERGY GENERATED (MWH) JNIT SERVICE FACTOR JNIT AVAILABILITY FACTOR JNIT CAPACITY FACTOR (USING MDC NET)	578261. 99.2 99.2 96.2	7632284. 7216779. 99.5 99.5 102.0	50306669 47186489 45 45
GROSS ELECTRICAL ENERGY GENERATED (MWH) NET ELECTRICAL ENERGY GENERATED (MWH) JNIT SERVICE FACTOR JNIT AVAILABILITY FACTOR JNIT CAPACITY FACTOR (USING MDC NET) JNIT CAPACITY FACTOR (USING DER NET)	578261. 99.2 99.2 96.2 94.9	7632284. 7216779. 99.5 99.5 102.0	50306669 47186489 45 45 44.1
GROSS ELECTRICAL ENERGY GENERATED (MWH) NET ELECTRICAL ENERGY GENERATED (MWH) JNIT SERVICE FACTOR JNIT AVAILABILITY FACTOR JNIT CAPACITY FACTOR (USING MDC NET)	578261. 99.2 99.2 96.2 94.9	7632284. 7216779. 99.5 99.5 102.0	50306669 47186489 45. 45. 44.

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-289

UNIT TMI-1
DATE 12-31-89

COMPLETED BY C.W. Smyth
TELEPHONE (717) 948-8551

MONTH: DECEMBER

DAY	OUERAGE DAILY POWER LEVEL (MWE-NET)	DAY AVE	RAGE DAILY POWER LEVEL (MWE-NET)
1	424.	17	798.
2	838.	18	792.
3	841.	19	775.
	843.	20	780.
4 5	845.	21	760.
5	845.	22	754.
7	845.	23	747.
8	849.	24	731.
9	848.	25	719.
10	845.	26	719.
11	844.	27	711.
12	845.	28	705.
13	843.	29	702.
14	844.	30	693.
15	822.	31	684.
16	803.		

REFUELING INFORMATION REQUEST

- 1. Name of Facility: Three Mile Island Nuclear Station, Unit 1
- 2. Scheduled date for next refueling shutdown: January 5, 1990 (8R)
- 3. Scheduled date for restart following refueling: March 5, 1990 (8R)
- 4. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment? No

If answer is yes, in general, what will these be?

If answer is no, has the reload fuel design and core configuration been reviewed by your Plant Safety Review Committee to determine whether any unreviewed safety questions are associated with the core reload (Ref. 10 CFR Section 50.59)? No

If no such review has taken place, when is it scheduled? January 15, 1990

- Scheduled date(s) for submitting proposed licensing action and supporting information: None Planned
- 6. Important licensing considerations associated with refueling, e.g. new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures: None
- 7. The number of fuel assemblies (a) in the core, and (b) in the spent fuel storage pool: (a) 177 (b) 360
- 8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned, in number of fuel assemblies:

The present licensed capacity is 752. Planning to increase licensed capacity through fuel pool reracking is in process.

9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity:

1991 is the last refueling discharge which allows full core off-load capacity (177 fuel assemblies).

UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH December 1989

DOCKET NO. 50-289
WII-I
DATE 12/31/89
COMPLETED BY C. W. Smyth
TELEPHONE 1717) 948-8551

No.	Date	Typel	Duration (Hours)	Reason?	Method of Shutting Down Reactor?	Licensee Event Repost #	System Cide 6 6	Component Cudes & 6	Course & Corrective Action to Prevent Recorrence
89-01	11/29/89	F	6.0	А	3				Unit trip was attributed to an EHC malfunction. Testing revealed a loose shield wire on the input from the primary speed sensor at TB150-3. The wire was tightened and no other loose connections were found.

F Forced S Scheduled

A Equipment Failure (Explain)
B. Maintenance of Test
C. Refueling
D. Regulatory Restriction
I Operator Training & License Examination
F. Administrative
G. Operational Error (Explain)
H. Other (Explain)

Method: 1 Manual 2 Manual Scrain. 3 Automatic Scrain. 4 Other (1 splain) Exhibit G - Instructions
for Preparation of Data
Entry Sheets for Locensee
Event Report (LER) File (NURFG0161)

Exhibit I Same Source

Actually used Exhibits F & H NUREC 0161