



Commonwealth Edison

Dresden Nuclear Power Station
R.R. #1
Morris, Illinois 60450
Telephone 815/942-2920

January 10, 1990

EDE LTR: #90-035

Director, Nuclear Reactor Regulation
United States Nuclear Regulatory Commission
Washington, DC 20555

Attention: Document Control Desk

Subject: Monthly Operating Data Report
Dresden Nuclear Power Station
Commonwealth Edison Company
Docket Nos. 50-010, 50-237, and 50-249

Gentlemen:

Enclosed is the Dresden Nuclear Power Station Monthly Operating Summary Report for December, 1989. This information is supplied to your office in accordance with the instructions set forth in Regulatory Guide 1.16. Please note that the report contains information which had been previously submitted to your attention on an annual basis in accordance with 10CFR50.59.

Sincerely,

E. D. Eenigenburg
Station Manager
Dresden Nuclear Power Station

EDE:GP:ae

Enclosure

cc: U.S. NRC Region III Office
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**MONTHLY NRC
SUMMARY OF OPERATING EXPERIENCE,
CHANGES, TESTS, AND EXPERIMENTS
PER REGULATORY GUIDE 1.16 AND 10 CFR 50.59
FOR
DRESDEN NUCLEAR POWER STATION
COMMONWEALTH EDISON COMPANY**

UNIT	DOCKET	LICENSE
1	050-010	DPR-2
2	050-237	DPR-19
3	050-249	DPR-25

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1.0 Introduction

Dresden Nuclear Power Station is a three reactor generating facility owned and operated by the Commonwealth Edison Company of Chicago, Illinois. Dresden Station is located at the confluence of the Kankakee and Des Plaines Rivers, in Grundy County, near Morris, Illinois.

Dresden Unit 1 is a General Electric Boiling Water Reactor with a design net electrical output rating of 200 megawatts electrical (MWe). The unit is retired in place with all nuclear fuel removed from the reactor vessel. Therefore, no Unit 1 operating data is provided in this report.

Dresden Units 2 and 3 are General Electric Boiling Water Reactors with design net electrical output ratings of 794 MWe each.

Waste heat is rejected to a man-made cooling lake using the Kankakee River for make-up and the Illinois River for blowdown.

The Architect-Engineer for Dresden Units 2 and 3 was Sargent and Lundy of Chicago, Illinois.

This report was compiled by Gerrine Paramore of the Dresden Technical Staff, telephone number (815)942-2920 extension 2364.

2.0 SUMMARY OF OPERATING EXPERIENCE FOR DECEMBER, 1989

2.1 UNIT 2 MONTHLY OPERATING EXPERIENCE SUMMARY

- 12-01-89 to 12-09-89 Unit 2 entered the month on line and operating at approximately 820 MWe.
- 12-10-89 to 12-21-89 On December 10, 1989 at approximately 0230 hours, Dresden Unit 2 began a planned outage to facilitate testing of the 125 VDC batteries. On December 20, 1989 at approximately 0710 hours the reactor was made critical. The generator was synchronized to the grid at 0821 hours on December 21, 1989.
- 12-22-89 to 12-25-89 On December 22, 1989 at 0300 hours the Unit 2 generator was separated from the grid and the unit shutdown to hot shutdown conditions due to a main generator casing liquid detector alarm indicating that seal oil was backing up into the generator casing. It was determined that a clogged seal oil float trap caused oil to overflow the seal oil drain enlargement resulting in oil backing up into the generator casing. The generator casing was drained of oil and the generator bushing boxes were opened to perform inspections. The clogged seal oil float trap and valve was cleaned of oil and debris. The generator was then synchronized to the grid at 0210 hours on December 25, 1989.
- 12-25-89 to 12-31-89 For the remainder of the month Dresden Unit 2 remained on line and operated as directed by the Corporate Load Dispatcher.

2.0 SUMMARY OF OPERATING EXPERIENCE FOR DECEMBER, 1989

2.2 UNIT 3 MONTHLY OPERATING EXPERIENCE SUMMARY

12-01-89 to 12-02-89 Unit 3 entered the month on line and operating at approximately 678 MWe.

12-03-89 to 12-31-89 On December 3, 1989 at 0313 hours, Dresden Unit 3 began its eleventh refueling outage and will be off line until February 10, 1990.

Major outage activities during the month included the following items:

- Disassembly of Unit 3 reactor vessel and removal of internals (separator and dryer)
- Local Leak Rate Testing
- Control Rod Drive removal and replacement
- Unload reactor fuel
- In-Service Inspection (ISI) program
- Detailed Control Room Design Review (DCRDR) human factors control panel modifications for Core Spray Systems A and B

3.0 OPERATING DATA STATISTICS

3.1 OPERATING DATA REPORT- UNIT TWO

DOCKET NO. 050-237
 UNIT DRESDEN TWO
 DATE JANUARY 1, 1990
 COMPLETED BY G. M. PARAMORE
 TELEPHONE (815)-942-2920

OPERATING STATUS

1. REPORTING PERIOD: DECEMBER 1989 GROSS HOURS IN REPORTING PERIOD: 744
 2. CURRENTLY AUTHORIZED POWER LEVEL (Mwt): 2,527 MAX DEPEND CAPACITY (MWe-Net): 772
 DESIGN ELECTRICAL RATING (MWe-Net) 794

3. POWER LEVEL TO WHICH RESTRICTED (IF ANY) (MWe-Net): N/A

4. REASONS FOR RESTRICTION (IF ANY):

REPORTING PERIOD DATA

	THIS MONTH	YEAR-TO-DATE	CUMULATIVE
5. TIME REACTOR CRITICAL (HOURS)	499.3	7,233.2	130,779.1
6. TIME REACTOR RESERVE SHUTDOWN (HOURS)	0.0	0.0	0.0
7. TIME GENERATOR ON-LINE (HOURS)	403.0	7,024.9	125,050.3
8. TIME GENERATOR RESERVE SHUTDOWN (HOURS)	0.0	0.0	0.0
9. THERMAL ENERGY GENERATED (MWh- <u>Gross</u>)	937,431	15,625,329	258,117,191
10. ELECTRICAL ENERGY GENERATED (MWh- <u>Gross</u>)	295,892	4,999,616	82,484,349
11. ELECTRICAL ENERGY GENERATED (MWh- <u>Net</u>)	279,575	4,745,977	77,988,883
12. REACTOR SERVICE FACTOR (%)	67.1	82.6	76.0
13. REACTOR AVAILABILITY FACTOR (%)	67.1	82.6	76.0
14. SERVICE FACTOR (%)	54.2	80.2	72.6
15. AVAILABILITY FACTOR (%)	54.2	80.2	72.6
16. CAPACITY FACTOR (USING MDC) (%)	48.7	70.2	58.7
17. CAPACITY FACTOR (USING DESIGN MWe) (%)	47.3	68.2	57.1
18. FORCED OUTAGE FACTOR (%)	15.0	2.7	10.8

19. SHUTDOWNS SCHEDULED OVER THE NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH):
 none

20. IF SHUTDOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP:
 n/a

3.0 OPERATING DATA STATISTICS

3.2 OPERATING DATA REPORT- UNIT THREE

DOCKET NO. 050-249
 UNIT DRESDEN THREE
 DATE JANUARY 1, 1990
 COMPLETED BY G.N. PARAMORE
 TELEPHONE (815)-942-2920

OPERATING STATUS

1. REPORTING PERIOD: DECEMBER 1989 GROSS HOURS IN REPORTING PERIOD: 744
 2. CURRENTLY AUTHORIZED POWER LEVEL (Mwt): 2,527 MAX DEPEND CAPACITY (MWe-Net): 773
 DESIGN ELECTRICAL RATING (MWe-Net) 794

3. POWER LEVEL TO WHICH RESTRICTED (IF ANY) (MWe-Net): N/A

4. REASONS FOR RESTRICTION (IF ANY):

REPORTING PERIOD DATA

	THIS MONTH	YEAR-TO-DATE	CUMULATIVE
5. TIME REACTOR CRITICAL (HOURS)	51.2	7,323.9	120,232.2
6. TIME REACTOR RESERVE SHUTDOWN (HOURS)	0.0	0.0	0.0
7. TIME GENERATOR ON-LINE (HOURS)	51.2	7,237.4	112,361.5
8. TIME GENERATOR RESERVE SHUTDOWN (HOURS)	0.0	0.0	0.0
9. THERMAL ENERGY GENERATED (MWh- <i>Gross</i>)	98,393	16,642,961	231,627,420
10. ELECTRICAL ENERGY GENERATED (MWe- <i>Gross</i>)	31,599	5,373,536	74,762,245
11. ELECTRICAL ENERGY GENERATED (MWe- <i>Net</i>)	24,244	5,117,954	70,855,387
12. REACTOR SERVICE FACTOR (%)	6.9	83.6	74.3
13. REACTOR AVAILABILITY FACTOR (%)	6.9	83.6	74.3
14. SERVICE FACTOR (%)	6.9	82.6	69.5
15. AVAILABILITY FACTOR (%)	6.9	82.6	69.5
16. CAPACITY FACTOR (USING MDC) (%)	4.2	75.6	56.7
17. CAPACITY FACTOR (USING DESIGN MWe) (%)	4.1	73.6	55.2
18. FORCED OUTAGE FACTOR (%)	0.0	3.1	12.1

19. SHUTDOWNS SCHEDULED OVER THE NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH):
 none

20. IF SHUTDOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP:
 FEBRUARY 10, 1990

3.3 AVERAGE DAILY UNIT POWER LEVEL (MWe-Net)

DOCKET NO. 450-237
 UNIT DRESDEN TWO
 DATE JANUARY 1, 1990
 COMPLETED BY G.R. PARAMORE
 TELEPHONE (815)-942-2920

REPORT PERIOD: DECEMBER 1989

DAY	POWER	DAY	POWER
1	782	17	-7
2	758	18	-7
3	766	19	-7
4	786	20	-9
5	772	21	174
6	784	22	94
7	784	23	-11
8	740	24	-11
9	660	25	405
10	1	26	614
11	-11	27	722
12	-7	28	757
13	-8	29	754
14	-8	30	723
15	-8	31	675
16	-8		

3.4 AVERAGE DAILY UNIT POWER LEVEL (MWe-Net)

DOCKET NO. 050-249
 UNIT DRESDEN THREE
 DATE JANUARY 1, 1990
 COMPLETED BY G.N. PARAMORE
 TELEPHONE (815)-942-2920

REPORT PERIOD: DECEMBER 1989

DAY	POWER	DAY	POWER
1	642	17	-8
2	591	18	-8
3	14	19	-8
4	-8	20	-10
5	-8	21	-9
6	-8	22	-10
7	-9	23	-11
8	-8	24	-11
9	-8	25	-8
10	-11	26	-8
11	-10	27	-8
12	-7	28	-8
13	-8	29	-8
14	-8	30	-8
15	-8	31	-8
16	-8		

3.5 UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 050-237
 UNIT NAME DRESDEN UNIT II
 DATE January 1, 1990
 COMPLETED BY G. Paramore
 TELEPHONE (815)942-2920

REPORT MONTH: DECEMBER, 1989

NO.	DATE	TYPE ¹	DURATION (HOURS)	REASON ²	METHOD OF SHUTTING DOWN REACTOR ³	LICENSEE EVENT REPORT #	SYSTEM CODE ⁴	COMPONENT CODE ⁵	CAUSE & CORRECTIVE ACTION TO PREVENT RECURRENCE
8	12-10-89	S	269.85	B	1	N/A	N/A	N/A	On December 10, 1989 Dresden Unit 2 began a planned outage to facilitate testing of the 125 VDC batteries.
9	12-22-89	F	71.17	H	1	N/A	TI	TRP	On December 22, 1989 Dresden Unit 2 was separated from the grid due to seal oil backing up into the generator casing. It was determined that a clogged seal oil float trap caused oil to overflow the hydrogen seal oil drain enlargement, resulting in oil backing up into the generator casing. The generator casing was drained and inspections were performed. The clogged seal oil float trap was cleaned of oil & debris. The generator was synchronized to the grid at 0210 hours on December 25, 1989.

¹
 F: Forced
 S: Scheduled

²
 Reason:
 A-Equipment Failure (Explain)
 B-Maintenance or Test
 C-Refueling
 D-Regulatory Restriction
 E-Operator Training & Licensee Examination
 F-Administrative
 G-Operational Error
 H-Other (Explain)

³
 Method:
 1-Manual
 2-Manual Scram
 3-Automatic Scram
 4-Other (Explain)
 5-Load Reduction

⁴
 Exhibit G-Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

⁵ Exhibit I - Same Source

3.6 UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 050-249
 UNIT NAME DRESDEN UNIT III
 DATE January 1, 1990
 COMPLETED BY G. Paramore
 TELEPHONE (815)942-2920

REPORT MONTH DECEMBER, 1989

No.	DATE	TYPE ¹	DURATION (HOURS)	REASON ²	METHOD OF SHUTTING DOWN REACTOR ³	LICENSEE EVENT REPORT #	SYSTEM CODE ⁴	COMPONENT CODE ⁵	CAUSE & CORRECTIVE ACTION TO PREVENT RECURRENCE
6	12-3-89	S	692.78	C	1	N/A	N/A	N/A	In the 11th refueling outage (through February, 1990)

- ¹
 F: Forced
 S: Scheduled
- ²
 Reason:
 A-Equipment Failure (Explain)
 B-Maintenance or Test
 C-Refueling
 D-Regulatory Restriction
 E-Operator Training & Licensee Examination
 F-Administrative
 G-Operational Error
 H-Other (Explain)

- ³
 Method:
 1-Manual
 2-Manual Scram
 3-Automatic Scram
 4-Other (Explain)
 5-Load Reduction

- ⁴
 Exhibit G-Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

- ⁵ Exhibit I - Same Source

COMMONWEALTH EDISON COMPANY - DRESDEN NUCLEAR POWER STATION

MAXIMUM DAILY ELECTRICAL LOAD FORM

FOR THE MONTH OF DEC, 1989

Day	Hour Ending	kWe
1	1600	1498200.
2	1700	1496700.
3	0100	1031700.
4	2200	825300.
5	2000	826900.
6	0500	824500.
7	0800	826000.
8	0100	812600.
9	1000	784800.
10	0100	167400.
11	0000	0.
12	0000	0.
13	0000	0.
14	0000	0.
15	0000	0.
16	0000	0.
17	0000	0.
18	0000	0.
19	0000	0.
20	0000	0.
21	2400	524200.
22	0100	525700.
23	0100	100.
24	0100	100.
25	2400	559400.
26	2000	703700.
27	2300	825000.
28	0100	819100.
29	1100	789000.
30	2100	788800.
31	2200	797700.
TOTAL		15426900.

4.0 UNIQUE REPORTING REQUIREMENTS

4.1 MAIN STEAM RELIEF VALVE OPERATIONS

Relief valve operations during the reporting period, December, 1989, are summarized in the following table. The table includes information as to which relief valve was actuated, how it was actuated, and the circumstances resulting in its actuation.

<u>Unit</u>	<u>Date</u>	<u>Valves</u> <u>Actuated</u>	<u>No. and Type</u> <u>of</u> <u>Actuations</u>	<u>Plant</u> <u>Conditions</u>	<u>Description</u> <u>of Events</u>
3	12/89	Safety Valve serial Nos: BK 7160 BK 6530	2, Manual	Refuel	These spare safety valves were rebuilt and bench tested for future use.

4.2 OFF-SITE DOSE CALCULATION MANUAL (ODCM) CHANGES

No ODCM changes were reported for the month of December 1989.

4.3 MAJOR CHANGES TO THE RADIOACTIVE WASTE TREATMENT SYSTEMS

There were no major changes to the radioactive waste treatment systems at Dresden during the reporting period.

4.4 FAILED FUEL ELEMENT INDICATIONS

4.4.1 Unit 2

Dresden Unit 2 fuel performance during December 1989 continued to show no indications of leaking fuel. This is based on the sum of the activities of the six noble gases as measured at the recombiner. Based on the reported data, Unit 2 had acceptable fuel performance.

4.4.2 Unit 3

Dresden Unit 3 fuel performance during December 1989 continued to show no indications of leaking fuel. This is based on the sum of the activities of the six noble gases as measured at the recombiner. Based on the reported data, Unit 3 had acceptable fuel performance.

5.0 PLANT OR PROCEDURE CHANGES, TESTS, EXPERIMENTS, AND SAFETY RELATED MAINTENANCE

5.1 Amendments to Facility License or Technical Specifications

The license amendments and/or Technical Specification changes which were approved and implemented for use during the reporting period are listed below.

5.1.1 Unit 2

None

5.1.2 Unit 3

None

5.2 Changes to Procedures Which are Described in the FSAR (Units 2 and 3)

Table 5.2.1, attached, summarizes the revisions to procedures described in the FSAR which were approved during the reporting period.

TABLE 5.2.1

CHANGES TO PROCEDURES WHICH ARE DESCRIBED IN THE FSAR (UNITS 2 AND 3)

PROCEDURE TYPE	PROCEDURE NO.	PROCEDURE TITLE/DESCRIPTION	SUMMARY OF CHANGES
Dresden Administrative Procedure (DAP)	DAP 7-5	Operating Logs and Records	2

- NOTES: 1. Administrative change; intent of procedure unchanged.
2. Changed for clarification, intent of procedure unchanged.
3. Changed to incorporate requirements for new equipment; intent of procedure unchanged
4. Changed to implement improved testing/calibration methodology; intent of procedure unchanged.

5.3 Significant Tests and Experiments Not Described in the FSAR
(Units 2 and 3)

Significant special procedures involving tests not described in the FSAR which were approved during the month are listed below.

<u>Procedure No.</u>	<u>Procedure Title/Description</u>
SP 89-12-124 SP 89-12-127	This procedure provided modification testing for Unit 3 3B Recirculation Motor Generator Set oil cooler bypass lines, which were installed in accordance with Modification M12-3-88-66.
SP 89-12-129	This procedure allowed for measuring the instrument response time for the Generator Load Reject circuitry. This information will be used to ensure that the values used for the safety reload analysis are accurate.
SP 89-12-137	This procedure delineated the steps and actions that were required to remove and install a jet pump beam working from the refuel bridge.
SP 89-12-140	This procedure was implemented to individually test five Control Rod Drive (CRD) Hydraulic Control Unit (HCU) 305-115 valves with the CRD HCU 301-25 valve out-of-service.
SP 89-12-145	This procedure provided modification testing for the Unit 3 3A Low Pressure Coolant Injection (LPCI) Heat Exchanger Shell Side Vent Modification.

5.4 Safety related maintenance (Units 2 and 3)

Safety related maintenance activities are summarized in the attached tables.

DRESDEN UNIT 2
5.4 SAFETY RELATED MAINTENANCE

EQUIPMENT	NATURE OF MAINTENANCE	LER OR OUTAGE NUMBER	CAUSE	MALFUNCTION RESULT	CORRECTIVE ACTION
MOV 1301-10	PREVENTIVE MR 079360	N/A			REMOVED AND INSTALLED ELECTRICAL COMPONENTS PER PROCEDURE
TORQUE WRENCH (08-9)	PREVENTIVE MR 081960	N/A			TESTED AND FOUND NO PROBLEMS
HPCI TURBINE	PREVENTIVE MR 083023	N/A			INSTALLED AND REMOVED TEST EQUIPMENT
2-1540-7 LPCI SYSTEM TOTAL FLOW	PREVENTIVE MR 084599	N/A			REPLACED HUMAN FACTORS SCALE
DW RAD MONITOR/TORUS RAD MONITOR	PREVENTIVE MR 084694	N/A			INSTALLED NEW HUMAN FACTORS SCALE
2-2406-B CHB POST LOC A H2 RECORDER	PREVENTIVE MR 084696	N/A			REPLACED HUMAN FACTORS SCALE
TORQUE WRENCH DW-9	CORRECTIVE MR 085066	N/A			REPLACED 6 MACHINE SCREWS ON SIGNAL CONVERTER AND TORQUED
2-2301-2 HPCI STEAM TRAP	CORRECTIVE MR 085599	N/A			CLEANED AREAS AND INSTALLED NEW GASKEY
LIS 2-263-72C REACTOR LOW WATER LEVEL EGGS YARWAY	CORRECTIVE MR 086681	N/A			REPLACED VENT VALVES AND TUBING CONNECTOR USING SEALANT

DRESDEN UNIT 3
SAFETY RELATED MAINTENANCE

EQUIPMENT	NATURE OF MAINTENANCE	LER OR OUTAGE NUMBER	MALFUNCTION CAUSE	RESULT	CORRECTIVE ACTION
3-6700 152-3433 3C SHUTDOWN COOLING PUMP BREAKER	CORRECTIVE NR D73231	N/A	-----	-----	INSTALLED NEW DISCONNECT
03-55 PANEL	CORRECTIVE NR D81173	N/A	-----	-----	REPLACED FAN MOTOR AND BLADE
LPRM 24-170	CORRECTIVE NR D86070	N/A	-----	-----	PERFORMED PROCEDURE AND CLEARED SPIKING
LPRM 3C-48-33	CORRECTIVE NR D86746	N/A	-----	-----	PERFORMED PROCEDURE TO CLEAR HIGH SPIKING
LPRM 40-570	CORRECTIVE NR D86784	N/A	-----	-----	PERFORMED PROCEDURE AND BURNED OFF WHISKERS
LPRM 16-250	CORRECTIVE NR D86784	N/A	-----	-----	PERFORMED PROCEDURE AND BURNED OFF WHISKERS
3-2419-B 3B DRYWELL HI RAD MON.	CORRECTIVE NR D86793	N/A	-----	-----	REPAIRED CABLE CONNECTORS PIN
LPRM 08-250	CORRECTIVE NR D86943	N/A	-----	-----	PERFORMED PROCEDURE AND REMOVED WHISKER BUILD UP
SRM #23	CORRECTIVE NR D86980	N/A	-----	-----	CLEANED EDGE CONNECTOR CONTACTS ON PULSE HEIGHT DISCRIMINATOR WITH CONTACT CLEANER
ANALOG TRIP SYSTEM DIV II 2202-73B TROUBLE	CORRECTIVE NR D87118	N/A	-----	-----	COMPLETED SETPOINT CHANGE PER PROCEDURE TO RAISE HPCI CARDS, ALARMS ARE NOW CLEAR

5.5 Completed Safety Related Modifications (Units 2 and 3)

Unit 2 and Unit 3 safety related modification packages closed during the month of December, 1989 are listed below. Only modifications which have been completely closed are listed; modifications which are authorized for use but not completely closed will be reported based on the date of their final closure. For ease of reference, the changes have been identified by their design change control modification number.

Modification No.

Description

None

No safety related modification packages were closed during the month of December, 1989.

5.6 Temporary System Alterations (Unit 2 and Unit 3)

A "Temporary System Alteration" refers to electrical jumpers, lifted leads, removed fuses, fuses turned to non-conducting position, fuses moved from normal to reserve holder, temporary power supplies, test switches in alternate positions, temporary blank flanges, and spool pieces. Alterations controlled and documented as part of a routine out-of-service or other procedure, alterations which are a normal feature of system design, and hoses installed as part of a venting or draining process are not included.

5.6.1 Unit 2

<u>Temporary System Alteration No.</u>	<u>Description</u>	<u>Installation Date</u>	<u>Removal Date</u>
II-81-89	Alteration to suspend a chain and buoy device from the grating floor in the Cribhouse to indicate water level in the Containment Cooling Service Water (CCSW) suction bay.	12-11-89	---
II-82-89	Alteration to remove recirculation system automatic controller from panel 902-4 to mask scales for Detailed Control Room Design Review (DCRDR) human factors control panel work. Alteration removed manual/automatic speed controllers to upgrade scales for DCRDR.	12-12-89	12-13-89
II-83-89	Alteration to remove manual/automatic speed controller on the "A" recirculation pump at panel 902-4 to upgrade the scales for the DCRDR work. The motor-generator (MG) set scoop tube was locked out to maintain a constant pump speed.	12-12-89	12-13-89

<u>Temporary System Alteration No.</u>	<u>Description</u>	<u>Installation Date</u>	<u>Removal Date</u>
II-84-89	Alteration to install a temporary hose from the discharge of deep well pump #1 to the well water supply piping, pending repairs to a frozen water main.	12-17-89	---
II-85-89	Alteration to install a jumper to defeat the high crankcase pressure switch on the Unit 2/3 Diesel Generator. The alteration maintained operability of the diesel generator (pending installation of a new crankcase pressure switch) without limiting ability to perform its emergency function.	12-19-89	---
II-86-89	Alteration to install jumper between terminal points RC and G on the PRIME computer recirculation fan to bypass a failed on/off switch pending repairs.	12/22/89	---

5.6.2 Unit 3

Temporary System Alteration No.	Description	Installation Date	Removal Date
III-48-89	Alteration to lift cable #76024 at terminal point WW-1 of panel 903-3, install a jumper from WW-7 to AA-44, lift cable #76027 at terminal point XX-1 and install jumper from XX07 to RR-34. This alteration was performed to support DCRDR work in accordance with Modification M12-3-86-24I.	12-4-89	----
III-50-89	Alteration to allow installation of jumpers to allow busses 35, 36, 37, 38 and 39 to be paralalled for breaker maintenance and relay calibration.	12-4-89	----
III-51-89	Alteration to disconnect present feed of Low Pressure Coolant Injection (LPCI) flow recorder 3-1540-7 and feed it from essential bus circuit 14, to facilitate installation of Modification M12-3-86-24A.	12-5-89	12-15-89
III-52-89	Alteration to install a blank flange on the alterrex bearing oil system of the main generator to facilitate maintenance activities and permit the operation of the oil system.	12-6-89	----
III-53-89	Alteration to temporarily lift cable 38734 at junction box 3TB-132. This disabled the carbon dioxide system to faciliate removal of Alterex housing to perform maintenance work on the main generator.	12-6-89	----

5.6.2 Unit 3 (Cont'd)

Temporary System Alteration No.	Description	Installation Date	Removal Date
III-54-89	Alteration to install a blank flange on the carbon dioxide supply line to prevent foreign material from getting into the piping during main generator maintenance work.	12-7-89	---
III-55-89	Alteration to install a three way jumper from terminal point AA-62 to AA-59 and relay 127 (pin 4) at panel 903-28. The purpose of this jumper is to bypass rod blocks, while all fuel is out of the reactor vessel to facilitate replacement of CRDs and perform maintenance work on Source Range Monitor (SRMs) and Intermediate Range Monitor (IRMs).	12-12-89	---
III-56-89	Alteration to install jumpers at terminals B-55 to B-56, B-57 to B-48 at panel 903-15, and B-57 to B-56, B-57 to B-58 at panel 903-17 to facilitate installation of modification M12-3-88-46. Placement of these jumpers prevented a Group II isolation from spurious drywell high radiation signals during the modification work. This alteration was permitted with the unit in a refuel outage with no fuel in the vessel.	12-12-89	---

5.6.2 Unit 3 (Cont'd)

Temporary System Alteration No.	Description	Installation Date	Removal Date
III-57-89	Alteration to provide an alternate feed for the annunciator windows E-19, F-19, A-17 and G-18 at panel 903-4 for installation of Modification M12-3-88-22B.	12-16-89	12-23-89
III-58-89	Alteration to replace the shutdown cooling and pool cooling temperature recorder (3-1040-2) at panel 903-4 with a newer model. Temporary indication of point 11 on the recorder (skimmer surge tank outlet temperature) was provided during the replacement.	12-18-89	12-22-89
III-59-89	Alteration to replace reactor conductivity recorder. This recorder indicates reactor water conductivity at the inlet and outlet of the cleanup demineralizers. During the replacement, local conductivity measurements were taken at the local rack 2203-2.	12-21-89	12-23-89