



Commonwealth Edison
LaSalle County Nuclear Station
Rural Route #1, Box 220
Marseilles, Illinois 61341
Telephone 815/357-6761

January 10, 1990

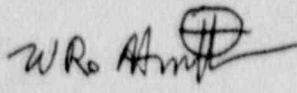
Director of Nuclear Reactor Regulation
United States Nuclear Regulatory Commission
Mail Station P1-137
Washington, D.C. 20555

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Gentlemen:

Enclosed for your information is the monthly performance report covering
LaSalle County Nuclear Power Station for December, 1989.

Very truly yours,


for G. J. Diederich
Station Manager
LaSalle County Station

GJD/JWT/sjj

 Enclosure

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LASALLE NUCLEAR POWER STATION

UNIT 1

MONTHLY PERFORMANCE REPORT

DECEMBER 1989

COMMONWEALTH EDISON COMPANY

NRC DOCKET NO. 050-373

LICENSE NO. NPF-11

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I. INTRODUCTION

The LaSalle County Nuclear Power Station is a two-unit facility owned by Commonwealth Edison Company and located near Marseilles, Illinois. Each unit is a Boiling Water Reactor with a designed net electrical output of 1078 Megawatts. Waste heat is rejected to a man-made cooling pond using the Illinois River for make-up and blowdown. The architect-engineer was Sargent and Lundy and the primary construction contractor was Commonwealth Edison Company.

Unit One was issued operating license number NPF-11 on April 17, 1982. Initial criticality was achieved on June 21, 1982 and commercial power operation was commenced on January 1, 1984.

This report was compiled by John W. Thunstedt, telephone number (815)357-6761, extension 2463.

II. MONTHLY REPORT

A. SUMMARY OF OPERATING EXPERIENCE

<u>Day</u>	<u>Time</u>	<u>Event</u>
1	0000	Reactor subcritical, generator off-line, L1R03 in progress.
31	2400	Reactor subcritical, generator off-line, L1R03 in progress.

B. PLANT OR PROCEDURE CHANGES, TESTS, EXPERIMENTS AND SAFETY RELATED MAINTENANCE.

1. Amendments to the Facility License or Technical Specification.

Removal of cycle-specific parameters

2. Changes to procedures which are described in the Safety Analysis Report.

LLP-89-058 "Loss of Power to Bus 242Y with Failure of D/G 2A to Start or Load"

This procedure provides a method of crosstying the "1A" D/G to Bus 242Y via the Unit-tie breakers 1424 and 2424 to provide backup emergency AC power to Bus 242Y via the "1A" D/G as required by Technical Specification during an extended "0" D/G outage with Unit 2 at power.

3. Tests and Experiments not described in the Safety Analysis Report. (None)

4. Major corrective maintenance to Safety-Related Equipment. (See Table 1)

5. Completed Safety-Related Modifications. (See Table 2)

B.4 TABLE 1

MAJOR CORRECTIVE MAINTENANCE TO
SAFETY-RELATED EQUIPMENT

WORK REQUEST NUMBER	COMPONENT	CAUSE OF MALFUNCTION	RESULTS AND EFFECTS ON SAFE PLANT OPERATION	CORRECTIVE ACTION
(Unit 0)	(None)			
(Unit 1)				
L94701	1AP-04E 0 D/G Output Breaker	Dirty Contact	Breaker would not close	Replaced HACR (control) relay
L94877	1E22-S001 HPCS D/G fuel-oil filter	Piping Crack	Oil Leak (spray)	Replaced fitting
L95610	Drywell Personnel Air-lock, Inner Door Drive Shaft	Seal Failure	Air leakage	Replaced seals

COMPLETED SAFETY-RELATED MODIFICATIONS

NUMBER	DESCRIPTION
M01-1-87-096	Replace the existing RPS and PCIS reactor water level switches with an analog trip system consisting of Rosemount level transmitters and trip units.
Temporary System Change 01-87-89	
	Installed a temporary Electrical Feed from Unit 2 Division I to Unit 1 Division I to provide power to the '0' D/G Immersion heater, AC Soakback pump, Oil Circulating pump, and Generator Space heater to allow these loads to perform as designed during the Unit 1 Division I Battery Modification.
Work Request L94283 "Locking the '1B' Diesel Generator Governor Run Solenoid for Troubleshooting"	
	Locking the governor run solenoid in the run position does not prevent the DG from performing its design function, however it will prevent a shutdown of the engine only, on differential overcurrent and the emergency stop button. This is because the engine trips are initiated by the lockout relay via the run solenoid. This will not affect the D/G output breaker and the engine. The overspeed trip is not affected because the overspeed is a total mechanical function that prevents the fuel injectors from supplying fuel. All other engine and breaker trips with the exception of differential overcurrent, overspeed, and emergency stop are bypassed in an ECCS Condition.

C. LICENSEE EVENT REPORTS

<u>LER Number</u>	<u>Date</u>	<u>Description</u>
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Correction to the previous (November 1989) report:

The event reported as LER #89-028-00 (dated 11/22/89) was determined not to be an LER prior to its submittal to the NRC. This number will be re-issued to the next reportable event to maintain sequential numbering.

89-028-00	12/04/89	Shutdown Cooling Outboard isolation valve auto-closure due to communication error during instrument surveillance.
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D. DATA TABULATIONS

1. Operating Data Report (attached)
2. Average Daily Unit Power Level (attached)
3. Unit Shutdowns and Power Reductions (attached)

D.1 OPERATING DATA REPORT

DOCKET NO. 050-373
 UNIT LASALLE ONE
 DATE January 10, 1990
 COMPLETED BY J.W. THUNSTEDT
 TELEPHONE (815)-357-6761

OPERATING STATUS

1. REPORTING PERIOD: DECEMBER 1989 GROSS HOURS IN REPORTING PERIOD: 744

2. CURRENTLY AUTHORIZED POWER LEVEL (MWt): 3,323 MAX DEMAND CAPACITY (MWe-Net): 1,036
 DESIGN ELECTRICAL RATING (MWe-Net): 1,078

3. POWER LEVEL TO WHICH RESTRICTED (IF ANY) (MWe-Net): 0

4. REASONS FOR RESTRICTION (IF ANY): Refueling (LIR03)

REPORTING PERIOD DATA

	THIS MONTH	YEAR-TO-DATE	CUMULATIVE
5. TIME REACTOR CRITICAL (HOURS)	0.0	6,114.8	32,134.1
6. TIME REACTOR RESERVE SHUTDOWN (HOURS)	0.0	0.0	1,641.2
7. TIME GENERATOR ON-LINE (HOURS)	0.0	6,103.8	31,418.3
8. TIME GENERATOR RESERVE SHUTDOWN (HOURS)	0.0	0.0	1.0
9. THERMAL ENERGY GENERATED (MWHt-Gross)	0	19,054,990	87,833,568
10. ELECTRICAL ENERGY GENERATED (MWHt-Gross)	0	6,422,787	29,068,541
11. ELECTRICAL ENERGY GENERATED (MWe-Net)	-7,605	6,157,173	27,732,702
12. REACTOR SERVICE FACTOR (%)	0.0	69.6	61.1
13. REACTOR AVAILABILITY FACTOR (%)	0.0	69.6	64.2
14. SERVICE FACTOR (%)	0.0	69.5	59.7
15. AVAILABILITY FACTOR (%)	0.0	69.5	59.7
16. CAPACITY FACTOR (USING MDC) (%)	-1.0	67.7	50.9
17. CAPACITY FACTOR (USING DESIGN MWe) (%)	-0.9	65.0	48.9
18. FORCED OUTAGE FACTOR (%)	0.0	1.4	10.1

19. SHUTDOWNS SCHEDULED OVER THE NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH):
 (None)

20. IF SHUTDOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP:
 January 05, 1990

D.2 AVERAGE DAILY UNIT POWER LEVEL (MWe-Net)

DOCKET NO. 050-373
UNIT LASALLE ONE
DATE January 10, 1990
COMPLETED BY J.W. THUNSTEDT
TELEPHONE (815)-357-6761

REPORT PERIOD: DECEMBER 1989

DAY	POWER	DAY	POWER
1	-10	17	-11
2	-10	18	-10
3	-10	19	-10
4	-10	20	-11
5	-10	21	-11
6	-10	22	-11
7	-10	23	-11
8	-10	24	-10
9	-10	25	-10
10	-10	26	-10
11	-10	27	-10
12	-10	28	-10
13	-10	29	-10
14	-10	30	-10
15	-10	31	-11
16	-10		

D.3 UNIT SHUTDOWNS AND POWER REDUCTIONS > 20%

YEARLY SEQUENTIAL NUMBER	DATE	TYPE F: FORCED S: SCHEDULED	DURATION (HOURS)	REASON	METHOD OF SHUTTING DOWN THE REACTOR OR REDUCING POWER	CORRECTIVE ACTIONS/COMMENTS
54	9/15/89	S	744	C	4	Refueling Outage (L1R03)

E. UNIQUE REPORTING REQUIREMENTS

1. Safety/Relief valve operations

DATE	VALVES ACTUATED	NO & TYPE ACTUATION	PLANT CONDITION	DESCRIPTION OF EVENT
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(None)

2. ECCS System Outages

OUTAGE NO.	EQUIPMENT	PURPOSE
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(Note: the year and unit numbers have been omitted from the Outage Number)

(Unit 0) (None)

(Unit 1)

1603	LPCS Spool Piece	Remove
1606	1DG024	Repair seat
1633	1E22-C002 (DG)	Trouble-shoot motor
1647	LPCS/LPCI	Perform LTS-800-104, 500-19
1676	1E12-F016A(RH)	Install containment-spray spool piece
1684	1B D/G	Replace motor
1692	1E12-F099A/B (RH)	Perform LOP-NB-01
1659	Div. I Drywell Spray	Install ILRT spool piece
1710	"0" D/G ODG01K	Check/replace crankcase pressure switch
1717	Head Spray	Install jumper piping
1721	RCIC Steamline Drain	Replace double block 1E51-F052/F053
1728	1DO014	Disassemble, clean
1744	EFCV, 1E12-F359B (RH)	Flush
1747	EFCV, 1E12-F359B (RH)	Repair
1751	1G33-F042	Repack
1753	1A D/G	Hydrolaze cooler
1754	1A D/G	(Various EM work)
1756	1E12-F024B (RH)	Perform LOP-RH-07

1767	1E12-F016A (RH)	ILRT
1777	U-1 ECCS Systems	ILRT
1789	Div-II RHR SW	(Administrative)
1801	Drywell Spray	Remove ILRT Spool
1809	1C RHR Cy Fill	Perform LCP-RH-16
1830	A RHR 24, 27	Shutdown-Cooling Operations

3. Changes to the Off-Site Dose Calculation Manual (None).
4. Major changes to Radioactive Waste Treatment Systems. (None)
5. Indications of Failed Fuel Elements. (None)

LASALLE NUCLEAR POWER STATION

UNIT 2

MONTHLY PERFORMANCE REPORT

DECEMBER, 1989

COMMONWEALTH EDISON COMPANY

NRC DOCKET NO. 050-374

LICENSE NO. NPF-18

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1. Amendments to Facility License or Technical Specifications
2. Changes to procedures which are described in the Safety Analysis Report.
3. Tests and Experiments not covered in the Safety Analysis Report.
4. Corrective Maintenance of Safety-Related Equipment
5. Completed Safety Related Modifications

C. LICENSEE EVENT REPORTS

D. DATA TABULATIONS

1. Operating Data Report
2. Average Daily Unit Power Level
3. Unit Shutdowns and Power Reductions

E. UNIQUE REPORTING REQUIREMENTS

1. Safety/Relief Valve Operations
2. ECCS System Outages
3. Off-Site Dose Calculation Manual Changes
4. Major Changes to Radioactive Waste Treatment System
5. Indications of Failed Fuel Elements

I. INTRODUCTION

The LaSalle County Nuclear Power Station is a two-unit facility owned by Commonwealth Edison Company and located near Marseilles, Illinois. Each unit is a Boiling Water Reactor with a designed net electrical output of 1078 Megawatts. Waste heat is rejected to a man-made cooling pond using the Illinois River for make-up and blowdown. The architect-engineer was Sargent and Lundy and the primary construction contractor was Commonwealth Edison Company.

Unit Two was issued operating license number NPF-18 on December 16, 1983. Initial criticality was achieved on March 10, 1984 and commercial power operation was commenced on June 19, 1984.

This report was compiled by John W. Thunstedt, telephone number (815)357-6761 extension 2463.

II. MONTHLY REPORT

A. SUMMARY OF OPERATING EXPERIENCE

Day	Time	Event
1	0000	Reactor critical, generator on-line at 3325 MWT.
9	2300	Reduced load to 2618 MWT for routine surveillances.
10	1900	Returned to full power.
13	0200	Reduced load to 1075 for CRD exercising.
	1000	Returned to full power.
17	0000	Reduced load to 2130 MWT for Heater-Bay and "A" Turbine-Driven Reactor Feed Pump repairs
18	1000	Returned to full power.
30	2100	Reduced load to 2034 MWT for rod-setting and steam leak repair.
	2300	Began slow ramp to full power.
31	1900	Steady at 3060 MWT, held per Load Dispatcher.
	2400	Reactor critical, generator on-line at 3060 MWT for Load Dispatcher.

B. PLANT OR PROCEDURE CHANGES, TESTS, EXPERIMENTS AND SAFETY RELATED MAINTENANCE.

1. Amendments to the Facility License or Technical Specification.
Removal of cycle-specific parameter limits.
2. Changes to procedures which are described in the Safety Analysis Report.
(None)

3. Tests and Experiments not described in the Safety Analysis Report.
(None)
4. Major corrective maintenance to Safety-Related Equipment.
(None)

Additionally, beginning this month, SOR switch failures will be reported separately from Nuclear Work Request reporting; see attached data sheets regarding the failure of 2B21-N038B and 2E22-N006.

5. Completed Safety-Related Modifications.
(See Table 2)

SOP dp Switch Failure Data Sheet

Equipment Piece Number: 2B21-N038B Model Number: 103-AS-B212-NX-JJTTX6

Serial Number: 88-10-4047

Application: Reactor Vessel Low Water Level 3 ADS Permissive Switch

Date and Time of Discovery: 11/27/89 0130 hours

Reactor Mode: 1 Power Level: 100%

Calibration Tolerance: 58.4-59.4*WC

Nominal Setpoint: 63.78*WC

Action Limits: <56.1 or >61.7*WC

Reject Limits: <56.0 or >61.8*WC

Technical Specification

Limit: <= 64.84*WC

As Found Setpoint: 62.0*WC

Date and Time of Return to Service: 11/27/89 2015 hours

Model Number of Replacement Switch: 103-AS-B212-NX-JJTTX6

Serial Number of Replacement Switch: 89-1-1815

Cause: Instrument Setpoint Drift

Corrective Action: Replaced low water level switch and inspection of failed switch found no abnormalities

Deviation Report: 1-2-89-069

SOR dp Switch Failure Data Sheet

Equipment Piece Number: 2E22-N006 Model Number: 103-AS-BB202-NX-JJTTX6

Serial Number: 86-9-2669

Application: High Pressure Core Spray Minimum Flow Switch

Date and Time of Discovery: 11/17/89 2005 hours

Reactor Mode: 1 Power Level: 100%

Calibration Tolerance: 23.6-22.4*WC

Nominal Setpoint: 23.0*WC

Action Limits: <21.3 or >24.7*WC

Reject Limits: <20.0 or >26.0*WC

Technical Specification

Limit: 9.7 +/- 1.85*WC

As Found Setpoint: 26.2*WC

Date and Time of Return to Service: 11/18/89 1325 hours

Model Number of Replacement Switch: 103-AS-BB202-NX-JJTTX6

Serial Number of Replacement Switch: 88-10-4044

Cause: Instrument Setpoint Drift

Corrective Action: Replaced switch and inspected failed switch with no abnormalities found

Deviation Report: 1-2-89-067

B.5 TABLE 2

COMPLETED SAFETY RELATED MODIFICATIONS

<u>MODIFICATION NUMBER</u>	<u>DESCRIPTION</u>
M01-2-86-002	Removal of the NSSS supplied racks and install high density free standing spent fuel racks for the Unit 2 Spent Fuel Storage Pool.
M01-2-86-057	Reslope the cross tie drain on line 2HG060A-6" to the suppression pool to improve flow and to allow proper drainage.

C. LICENSEE EVENT REPORTS

<u>LER Number</u>	<u>Date</u>	<u>Description</u>
89-018-00	12/16/89	250 VDC Battery Electrolyte Temperature Low
89-019-00	12/26/89	RCIC High-Level Trip Switch Failure.

D. DATA TABULATIONS

1. Operating Data Report (Attached)
2. Average Daily Unit Power Level (Attached)
3. Unit Shutdowns and Power Reductions (Attached)

D.1 OPERATING DATA REPORT

DOCKET NO. 050-374
 UNIT LASALLE TWO
 DATE January 10, 1990
 COMPLETED BY J.W. THUNSTEDT
 TELEPHONE (815)-357-6761

OPERATING STATUS

1. REPORTING PERIOD: DECEMBER 1989 GROSS HOURS IN REPORTING PERIOD: 744

2. CURRENTLY AUTHORIZED POWER LEVEL (MWt): 3,323 MAX DEPEND CAPACITY (MWe-Net): 1,036
 DESIGN ELECTRICAL RATING (MWe-Net) 1,078

3. POWER LEVEL TO WHICH RESTRICTED (IF ANY) (MWe-Net): (None)

4. REASONS FOR RESTRICTION (IF ANY): (N/A)

REPORTING PERIOD DATA

	THIS MONTH	YEAR-TO-DATE	CUMULATIVE
5. TIME REACTOR CRITICAL (HOURS)	744.0	6,669.1	30,147.1
6. TIME REACTOR RESERVE SHUTDOWN (HOURS)	0.0	0.0	1,716.9
7. TIME GENERATOR ON-LINE (HOURS)	744.0	6,592.1	29,683.1
8. TIME GENERATOR RESERVE SHUTDOWN (HOURS)	0.0	0.0	0.0
9. THERMAL ENERGY GENERATED (MWh-Gross)	2,412,840	20,380,440	86,320,048
10. ELECTRICAL ENERGY GENERATED (MWh-Gross)	825,671	6,747,588	28,450,926
11. ELECTRICAL ENERGY GENERATED (MWe-Net)	797,050	6,496,055	27,235,305
12. REACTOR SERVICE FACTOR (%)	100.0	75.9	66.1
13. REACTOR AVAILABILITY FACTOR (%)	100.0	75.9	69.8
14. SERVICE FACTOR (%)	100.0	75.0	65.1
15. AVAILABILITY FACTOR (%)	100.0	75.0	65.1
16. CAPACITY FACTOR (USING NDC) (%)	103.4	71.4	57.6
17. CAPACITY FACTOR (USING DESIGN MWe) (%)	99.4	68.6	55.4
18. FORCED OUTAGE FACTOR (%)	0.0	14.2	16.3

19. SHUTDOWNS SCHEDULED OVER THE NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH):

Refueling (L2R03) 03-17-90 12 Weeks

20. IF SHUTDOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP:

(N/A)

D.2 AVERAGE DAILY UNIT POWER LEVEL (MWe-Net)

DOCKET NO. 050-374

UNIT LASALLE TWO

DATE January 10, 1990

COMPLETED BY J.W. THUNSTEDT

TELEPHONE (815)-357-6761

REPORT PERIOD: DECEMBER 1989

DAY	POWER	DAY	POWER
1	1,105	17	875
2	1,101	18	1,077
3	1,101	19	1,092
4	1,105	20	1,077
5	1,103	21	1,088
6	1,093	22	1,081
7	1,103	23	1,089
8	1,103	24	1,091
9	1,097	25	1,093
10	1,008	26	1,090
11	1,097	27	1,072
12	1,095	28	1,063
13	1,086	29	1,063
14	1,098	30	1,012
15	1,097	31	915
16	1,041		

D.3 UNIT SHUTDOWNS AND POWER REDUCTIONS >20%

YEARLY SEQUENTIAL NUMBER	DATE	TYPE F: FORCED S: SCHEDULED	DURATION (HOURS)	REASON CODE	METHOD OF SHUTTING DOWN THE REACTOR OR REDUCING POWER	CORRECTIVE ACTIONS/COMMENTS
33	9	S	0.0	B	5	Miscellaneous routine surveillances.
34	13	S	0.0	B	5	CRD exercising
35	17	S	0.0	B	5	Heater-bay and "A" TDRFP repairs
36	30	S	0.0	B	5	Rod-set and steam leak repair.

E. UNIQUE REPORTING REQUIREMENTS

1. Safety/Relief Valve Operations

DATE	VALVES ACTUATED	NO & TYPE ACTUATIONS	PLANT CONDITION	DESCRIPTION OF EVENT
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(None)

2. ECCS System Outages

OUTAGE NO.	EQUIPMENT	PURPOSE
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NOTE: The year and unit designators have been omitted from the Outage Number.

1040	2B RHR Pump	Lubricate, replace oil
1041	2A D/G	Clean Strainers
1042	HPCS Pump	Replace oil
1053	2A RHR Pump	Replace oil
1055	2B D/G	Replace 2E22-N504
1056	2B D/G	Lubricate
1057	2B D/G Motor- Driven Compressor	Replace high-pressure vent assembly
1061	2E22-C302A (DG)	Replace head gasket
1068	2DG058	Check "B" Compressor Operation
1069	2A D/G "A" Air Compressor	Repair oil leak

3. Changes to the Off-Site Dose Calculation Manual
(None).

4. Major changes to Radioactive Waste Treatment Systems.
(None)

5. Indications of Failed Fuel Elements.
(None)