Docket Nos: 50-460 513

APPLICANT: Washington Public Power Supply System

FACILITY: WMP-1/4

SUBJECT: Summary of Meeting with Washington Public Power Supply

System and Babcock and Wilcox to Discuss Inadequate Core

Cooling Instrumentation - July 29, 1981

Background

NUREG-0737 was issued in Movember 1980 and represents an ERC staff clarification of TMI-2 approved action plan requirements. On May 13, 1981 a proposed rule was published in the Federal Register to codify that portion of the NUREG-0737 applicable to operating license applications into the Commission's regulations (i.e. ICCFR 50). This proposed rule is presently being reviewed prior to becoming a final rule. Section 50.34(f) (2)(VII) of the proposed rule (i.e. itel II.F.Z of NUREG-0737) will require instrumentation to detect inadequate core cooling. At the request of the Mashington Public Power Supply System (MPPSS) a meeting was held at the MRC's Bethesda, Maryland offices for the purpose of allowing MPPSS and the MSSS supplier for the MMP-1 and MMP-4 projects (Babcock and Milcox) to present the conceptual design intended for use at these projects to meet the requirements of NUREG-0737 Item II.5.2. This meeting was held on July 29, 1981.

Enclosure I lists the meeting attendees.

The agenda and visual aids used in the meeting are enclosed as Enclosure 2.

Discussion

The representatives of MPPSS (hereafter called the Supply System) included in the opening statements the fact that the design discussed at the meeting applied only to the WNP-1/4 projects and did not represent a Babcock and Wilcox (B&W) generic design to be applied to other plants of B&W design. The Supply System also stated that the anticipated outcome of the meeting was MRC staff concurrence with the basic design and approach as a prelude to dedicating additional resources to detailed development of the design.

Babcock and Wilcox (Schieck) summarized what ask and the Supply System understand to be the requirements/comments of Item II.F.2. (see pages 2 through 5 of Enclosure 2). By defining inadequate core cooling (ICC) as "insufficient reactor core heat removal to preclude continuing fuel clad temperature increase to the point of substantial fuel damage", Baw took the position that the most reliable indications of the approach or onset of ICC are the core thermocouple

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readings. Consequently, one important feature that was emphasized is qualification of all incore thermocouples as safety-related equipment. The Supply System intends to qualify all incore thermocouples as required and to designate four thermocouples in each core quadrant to readout in a backup display. These requirements are detailed in attachment I to Item II.F.2.

The next portion of the presentation dealt with the applicability of various core parameters and corresponding instrumentation to the various core conditions from normal operation to complete core uncovery. This is shown in page 7 of Enclosure 2. A summary of RCS level vs volume demonstrated that by the time water level has dropped to a level corresponding to the bottom of the hot leg nozzles, 80% of the RCS inventory has been lost (assuming the pressurizer has drained). By comparison, with water level at 10 feet below the top of the hot leg piping (the top of the hot leg being the high point in the RCS) only 14% of the RCS has been lost assuming the pressurizer is empty (3%, with the pressurizer not empty). On this basis, the Supply System proposed use of differential pressure type level indicators on the not leg piping as an alternative to monitoring actual reactor vessel leve!. Item II.F.2 states that "the evaluation is to include reactor water level indication." The design proposed by the Supply System would therefore cover a range encompassing 80% of the inventory of the RCS but would not show levels below the hot leg nozzle. The point at which fuel begins to be uncovered corresponds to an 84% loss of RCS inventory at which time incore thermocouples would clearly indicate inadequate core cooling. Therefore only a "dead band" of 4% RCS inventory would exist between the low range of hot leg level indication and commencement of core uncovery. (See pages 8 and 9 of Enclosure 2).

A "margin-to-saturation" meter, display and alarm was proposed as part of the planned ICC instrument package and would have a range of between 200 F subcooled to 35 F superheated. An alarm would be provided at an appropriate setpoint to give advance warning of the approach to ICC.

Details of the various instruments comprising the ICC package were discussed including the inputs, outputs, number of channels, ranges and power supplies (page 10 through 14 of Enclosure 2). A tentative schedule of key steps in the development of the ICC package and a summary of the features of the proposed system and steps required for final acceptance were also discussed.

The staff raised the question of what system testing and qualification steps were planned in addition to environmental qualification of the individual components. The Supply System stated that only environmental testing/qualification was planned. The staff requested an evaluation be made to determine what type of testing should be performed to verify performance of the system design. As an example, it was pointed out that the Westinghouse reactor vessel level instrumentation system developed for this purpose is being tested as a system both at the vendor's facility and at the Semiscale Test Facility in Idaho (INEL). The staff offered to pursue the availability of Semiscale for testing of the system proposed for MPPSS, if necessary.

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A discussion on primary and backup displays relative to 99% computer availability ensued. The Supply System reiterated that they felt the proposal met those requirements of Item II.F.2.

Although the Supply System stated repeatedly that this proposal was unique only to the MNP-1 and WNP-4 units, the staff took a firm position with Babcock and Wilcox that the ICC instrumentation proposed at this meeting should be further developed into a generic solution to the extent possible for all B&W plants of similar design. Although this position was clearly stated and restated by the staff, no commitment was agreed to by Babcock and Wilcox.

Summary

Babcock and Wilcox proposed, on behalf of the Supply System, a combination of qualified incore thermocouples, margin-to-saturation instrumentation and hot leg/upper reactor vessel water level instrumentation to meet the ICC instrumentation requirements of Item II.F.2 in NUREG-0737 for WNP-1/4.

The staff responded that, with proper design, the level measurement concept proposed would probably be acceptable to the staff.

The Supply System was requested to evaluate additional testing and qualification of the proposed hot leg level indication system. It was also requested by the staff that this system be considered for submittal by B&W as a generic package to meet the ICC instrumentation requirements.

p/

Ronald W. Hernan, Project Manager Licensing Branch #4 Division of Licensing, NRR

Enclosures: As stated

cc: See next page

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NRC FORM	318 (10/80) NRCM	0240		RECORD C		# USGPO 1980329-824

A discussion ensued relative to primary and backup displays relative to 99% computer availability. The Supply System reiterated that they felt the proposal met those requirements of Item II.F.2.

Although the Supply System stated repeatedly that this proposal was unique only to the WNP-1 and WNP-4 units, the staff took a firm position with Babcock and Wilcox that the ICC instrumentation proposed at this meeting should be further developed into a generic solution to the extent possible for all B&W plants of similar design. Although this position was clearly stated and restated by the staff, no commitment was agreed to by Babcock and Wilcox.

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Ronald W. Hernan, Project Manager Licensing Branch #4 Division of Licensing, NKK

Enclosures: As stated

cc: See next page

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Resident Inspector/WPPSS NPS c/o U.S. Nuclear Regulatory Commission P.O. Box 69 Richland, Washington 99352

MEETING ATTENDEFS

July 29, 1981

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T. Huang, CPB H. Li, ICSB

R. Hernan, LPM/DL

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G. Schieck

R. Ganther

C. Mahaney

G. Andersen

R. Queenan

TENNESSEE VALLEY AUTHORITY

D. Wilson

UNITED ENSINEERS & CONSTRUCTORS

B. Hale

CONSUMERS POWER COMPANY

T. Sullivan

R. Huston

AGENDA FOR SUPPLY SYSTEM/NRC MEETING ON INADEQUATE CORE COOLING

1.	INTRODUCTION, NEED FOR NRC INDICATION OF APPROVAL OF PROPOSED CHANGES.	AG HOSLER
2.	PEQUIREMENTS FOR INSTRUMENTATION FOR THE DETECTION OF INADEQUATE CORE COOLING (ICC) IN NURES 0737.	GC SCHIECK
3.	DEFINITION OF ICC AS IT APPLIES TO NURES 0737.	GC SCHIECK
	INDICATION OF THE APPROACH AND ONSET OF ICC. 1CC INSTRUMENTATION PROVIDED ON MNP 1/4.	GC SCHIECK, GE ANDERSON RM QUEENAN
	1. RC SATURATION METER 2. EXTENDED RANGE THOT 3. RCS Level Indication (WR, NR, URV) 4. Incore Thermocouple Indication (sixteen planned to have IE READOUT)	
6.	HARDWARE JUSTIFICATION AND ANALYSIS TO SUPPORT ICC INSTRUMENTATION DESIGN REQUIRED BY NURES 0737.	GC SCHIECK
7.	Analyses planned for Operating Procedure Develop- ment	GC SCHIECK
8.	SUMMARY OF ICC INSTRUMENTATION PROVISIONS AND COMMITMENTS	GC SCHIECK, RL WEBRING
9.	NRC PRESENT THEIR OBSERVATIONS AND CONCURRENCE	NRC STAFF

SUUUABY_OE_SUPPLY_SYSTEU_ICC_COMMITMENTS

PROVIDE:

RC SATURATION METER
NARROW RANGE RC LEVEL INDICATION
WIDE RANGE RC LEVEL INDICATION
INCORE THERMOCOUPLE GUALIFICATION
INCORE THERMOCOUPLE GUALIFIED READOUT FOR
16 THERMOCOUPLES

PERFORM:

ANALYSIS TO DEFINE OPERATOR USE OF INSTRUMENTATION
PROVIDED
ANALYSIS TO CORRELATE INCORE THERMOCOUPLE READINGS
TO FUEL CLAD TEMPERATURES
HUMAN FACTORS ANALYSIS OF DISPLAYS AND ALARMS

REQUIREMENTS FOR INSTRUMENTATION FOR THE DETECTION OF INADEQUATE CORE COOLING (NUREG 0737, ITEM II.F.2)

- O PROVIDE UNAMBIGUOUS INDICATION OF ICC
- O SHOW THAT THE EXISTENCE OF INADEQUATE CORE
 COOLING CAUSED BY VARIOUS PHENOMENA IS INDICATED
- O SHOW THAT ADVANCED WARNING OF ICC IS GIVEN
- O SHOW THAT THE FULL RANGE FROM NORMAL OPERATION TO COMPLETE CORE UNCOVERY IS COVERED
- 6 SHOW THAT INADEQUATE CORE COOLING IS NOT ERRONEOUSLY INDICATED
- O PROVIDE NECESSARY ANALYSIS TO SUPPORT DESIGN
 OF PROPOSED INSTRUMENTATION
- @ HUMAN-FACTORS ANALYSIS OF TYPE AND LOCATION OF DISPLAYS AND ALARMS

HARDWARE JUSTIFICATION AND ANALYSIS TO SUPPORT ICC INSTRUMENTATION DESIGN

- DEMONSTRATE THAT THE DISPLAY AND ASSOCIATED HARDWARE BEYOND THE ISOLATION DEVICE FROM THE COMPUTER (IF USED) IS ENERGIZED BY A BATTERY BACKED SOURCE.
- O COMPUTER PROCESSED SIGNALS MUST BE SHOWN TO BE 99% AVAILABLE, HAVE POST ACCIDENT ACCESSIBLITY FOR MAINTENANCE, AND HAVING COMPLETELY QUALIFIED DIVERSE METHODS OF ICC MONITORING.
- 6 INCLUDE RV WATER LEVEL INDICATION IN EVALUATION.
- O EVALUATE CONFORMANCE OF INCORE THERMOCOUPLES WITH "DESIGN AND QUALFICATION CRITERIA FOR PWR INCORE THERMOCOUPLES".
- O EVALUATION FOR CONFORMANCE OF ICC MONITORING System to Design and Qualfication Criteria.
- @ DESCRIPTION OF OPERATING PROCEDURES.
- O DESCRIPTION OF ANALYSIS USED IN DEVELOPING PROCEDURES.

DEFINITION OF "INADEQUATE CORE COOLING" AS USED IN NUREG 0737

INSUFFICIENT REACTOR CORE HEAT REMOVAL TO PRECLUDE CONTINUING FUEL CLAD TEMPERATURE INCREASE TO THE POINT OF SUBSTANTIAL FUEL DAMAGE.

Approach of ICC Indicated BY:

Loss of Subcooling Margin
Loss of RC Inventory
Increase in Fuel Clad Temperatures

ONSET OF ICC INDICATED BY:

EXCESSIVE FUEL CLAD TEMPERATURES

INSTRUMENTATION FOR DETECTION OF THE APPROACH AND ONSET OF INADEQUATE CORE COOLING

	PARAMETER	NORMAL OPERATION	NATURAL CIRCULATION	LOSS OF		CORE UNCOVERY .
	RC PRESSURE	PRESSURIZER PRESSURE				
	RC FLOW	RC FLOW METER -		_		_
	RC HOT LEG TEMPERATURE	Тнот				
	SATURATION MARGIN	SATURATION METER				
	RCS	PRESSURIZER LEVEL		NARROW RANGE LEVEL	WIDE RANGE LEVEL	
	FUEL CLAD TEMPERATURE			INCORE THERMOCOUR	LES	
ICATES NEW OR	STATION	RCS			1	
		The state of the s	10' BELO HOT		BOTTOM	

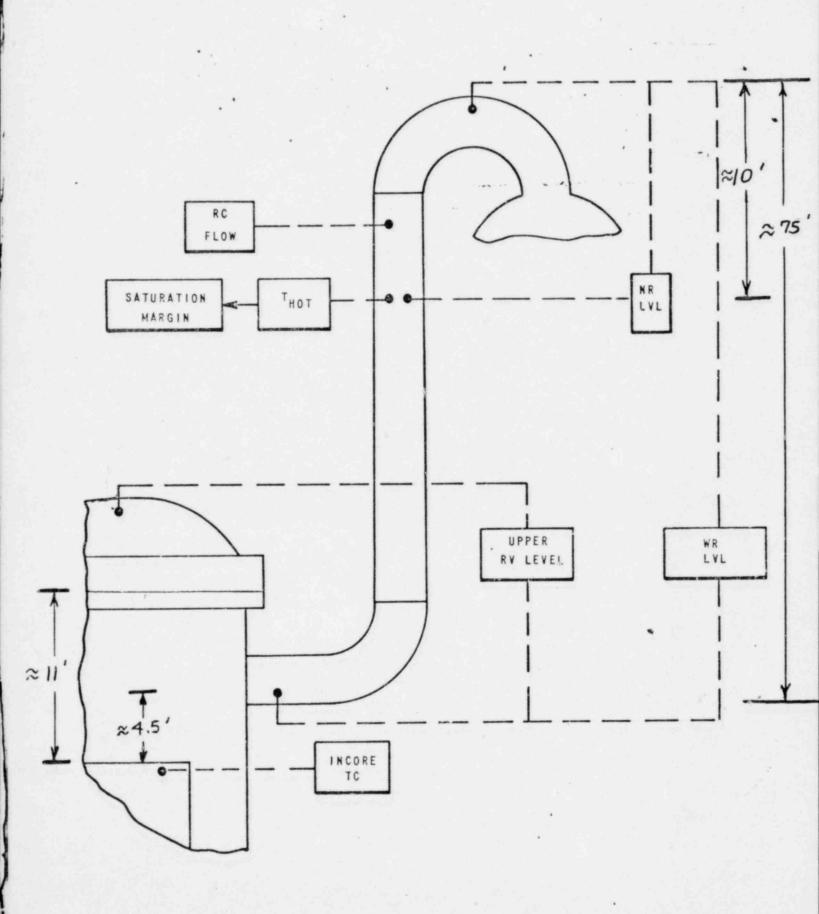
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RCS LEVEL VS. VOLUME CORRELATION

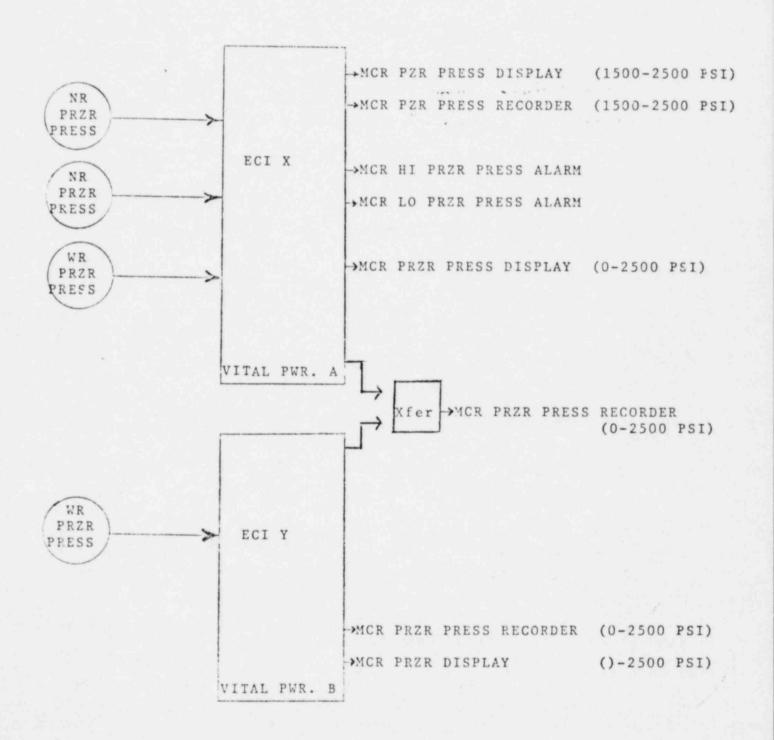
TOTAL RCS LIQUID VOLUME FOR 205 FA PLANT = 11.870 FT^3 EXCL. PZR LIQ. = 13.160 FT^3 INCL. PZR LIQ.

		TOTAL AMOUNT (FT ³) LOST IS	% OF RCS LIQU ON 11,870	% OF RCS LIQUID LEVEL LOSS (BASED ON 11,870 FT ³) IS		
IF RCS LEVEL DROPS TO	PZR DOES NOT DRAIN	PZR DRAINS	PZR DOES NOT DRAIN	PZR DRAINS		
BOTTOM OF NARROW RANGE HOT LEG LEVE	330 EL	1,620	3	14		
BOTTOM OF WIDE RANGE HOT LEG LEVE	8,210 EL	9,500	69	80		
TO TOP OF CORE	8,720	10,010	73	84		
		WITH LIQUID AT TOP (OF CORE			
REMAINING IN RV REMAINING IN LOOPS	2,350 FT ³ 800 FT ³			NAL TOTAL (11,870 NAL TOTAL (11,870	_	

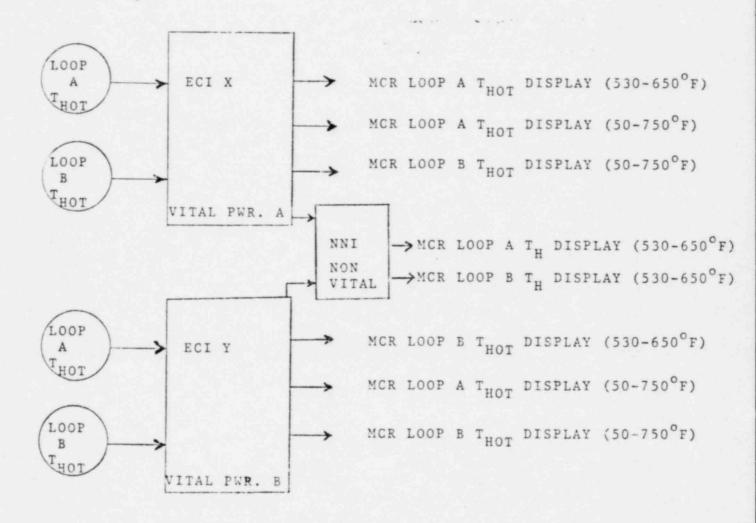
INSTRUMENTATION FOR DETECTION OF THE APPROACH . AND ONSET OF INADEQUATE CORE COOLING



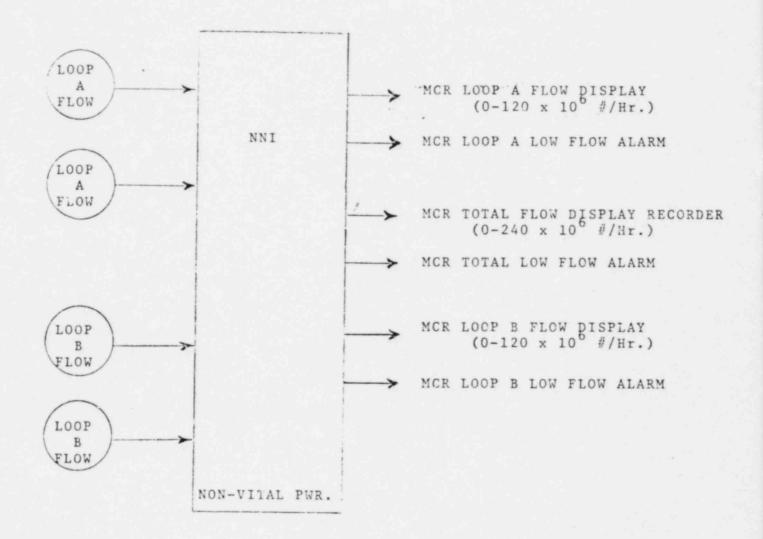
RC PRESSURE DISPLAY
(Protection System Equipment not Shown)



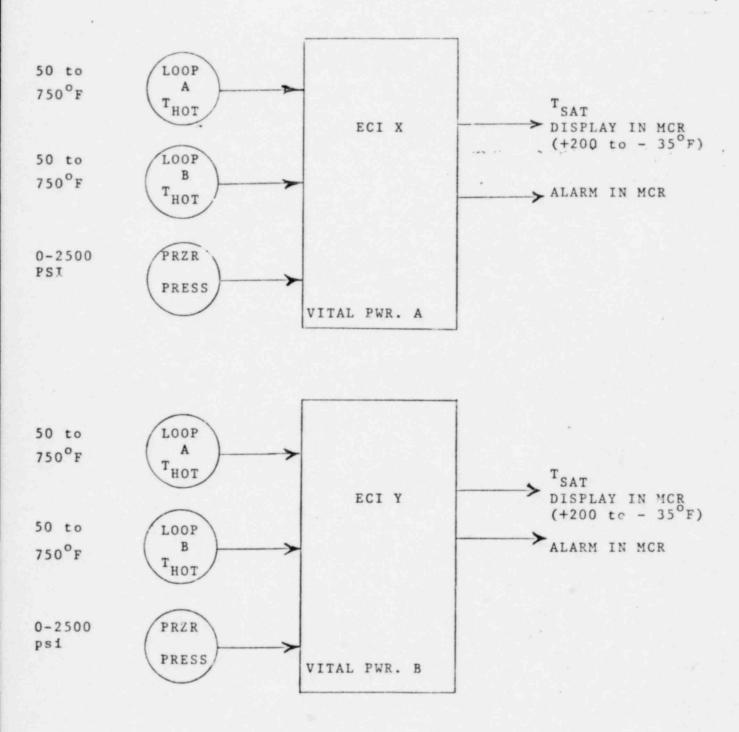
R.C. HOT LEG TEMPERATURE DISPLAY (Protection System Equipment not Shown)



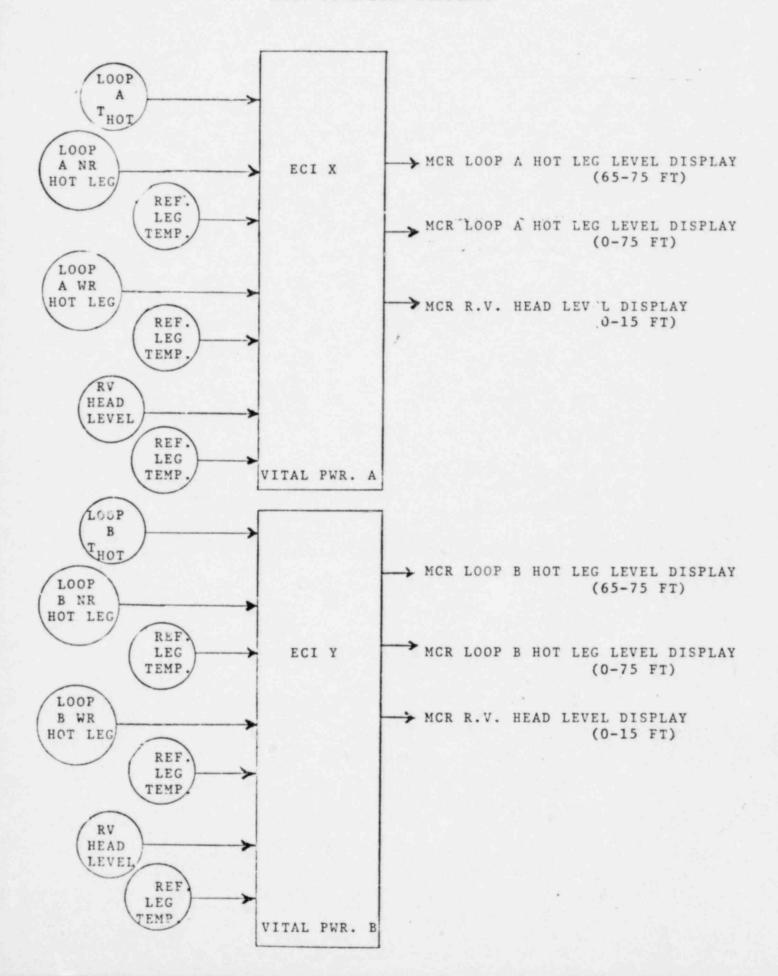
R.C. FLOW DISPLAY AND ALARM (Protection System Equipment not Shown)



MARGIN-TO-SATURATION METER DISPLAY AND ALARM



R.C. INVENTORY DISPLAYS



ADDITIONAL HARDWARE & ANALYSES TO SUPPORT INSTRUMENTATION FOR DETECTION OF ICC

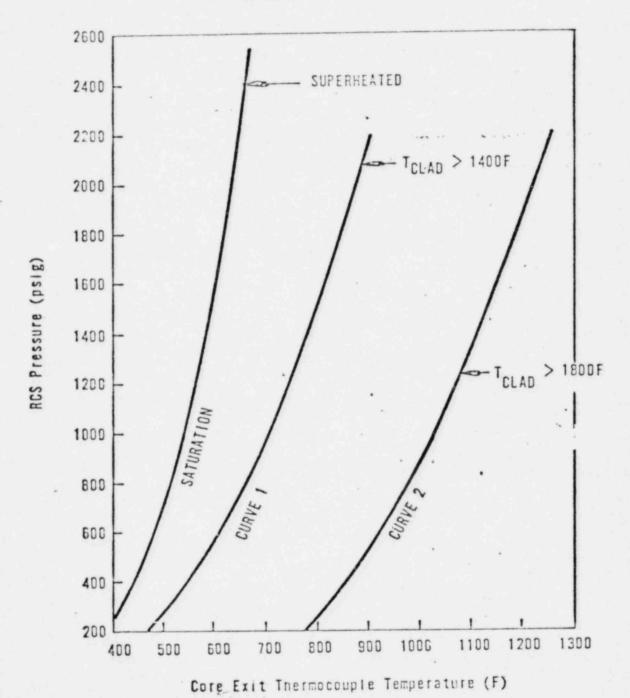
HARDWARE	COMPLETE HARDWARE DESIGN	HARDWARE SHIP
RCS LEVEL	4/82	10/33
EXTENDED RANGE THOT	2/82	3/83
RC SATURATION METER	2/32	8/82
INCORE THERMOCOUPLE QUALIFICA-		
TION	1	
INCORE THERMOCOUPLE DISPLAY		
ANALYSES	STARI	ESTIMATED COMPLETE
INCORE THERMOCOUPLE UTILIZA-	7/81	1/32
ANTICIPATED TRANSIENT OPERATING GUIDELINES (ATOG)	6/81	3/83
HUMAN FACTORS ANALYSIS OF DISPLAYS AND ALARMS	-	

ANALYSIS PLANNED FOR OPERATING PROCEDURE DEVEOPMENT

ANTICIPATED TRANSIENTS OPERATING GUIDELINES (ATOG)

- BASE ANALYSIS
- SBLOCA SUPPLEMENT
- ICC GUIDELINE SUPPLEMENT

177 FA PLANT
CORE EXIT FLUID TEMPERATURE FOR
INADEQUATE CORE COOLING



- INCORE THERMOCOUPLES PROVIDE UNAMBIGUOUS INDICATION OF ICC
- 1 ICC CAUSED BY VARIOUS PHENOMENA IS INDICATED
- ADVANCED WARNING OF ICC IS GIVEN
- THE FULL RANGE FROM NORMAL OPERATION TO COMPLETE CORE UNCOVERY IS COVERED
- 1 ICC IS NOT ERRONEOUSLY INDICATED
- NECESSARY ANALYSIS TO SUPPORT THE DESIGN OF PROPOSED INSTRUMENTATION IS PLANNED OR IN PROGRESS
- A HUMAN FACTORS ANALYSIS OF THE TYPE AND LOCATION OF DISPLAYS AND ALARMS WILL BE PERFORMED

- O COMPUTER BASED SIGNALS ARE NOT USED FOR PRIMARY ICC INDICATIONS
- RV WATER LEVEL INDICATION HAS BEEN EVALUATED AND UPPER RV LEVEL INDICATION WILL BE PROVIDED
- DEVELOPMENT OF AN INCORE THERMOCOUPLE QUALIFICATION.

 PROGRAM AND DISPLAY DESIGN MEETING THE REQUIREMENTS

 OF "DESIGN AND QUALIFICATION CRITERIA FOR PNR INCORE

 THERMOCOUPLES" IS IN PROGRESS
- A PROGRAM TO PROVIDE CONFORMANCE OF THE ICC MONITORING
 SYSTEM TO DESIGN AND QUALIFICATION CRITERIA IS IN
 PROGRESS
- A DESCRIPTION OF OPERATING PROCEDURES WILL BE PROVIDED
- A DESCRIPTION OF THE ANALYSIS USED TO DEVELOP ICC OPERATING
 PROCEDURES WILL BE PROVIDED

MEETING SUMMARY DISTRIBUTION

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