

APPENDIX A

NOTICE OF VIOLATION

Florida Power Corporation
Crystal River 3
Nuclear Generating Plant

Docket No. 50-302
License No. DPR-72

As a result of the inspection conducted on March 25 - April 28, 1981, and in accordance with the Interim Enforcement Policy, 45 FR 66754 (October 7, 1980), the following violations were identified.

- A. 10 CFR Part 50, Appendix B, Criterion XI requires that testing required to demonstrate that systems will perform satisfactorily in service is performed by written test procedures which incorporate the requirements and acceptance limits contained in applicable design documents.

The licensee's Quality Program, as delineated in Section 1.7.6.1.7.k requires test procedures to incorporate or reference the requirements and acceptance limits contained in applicable design and procurement documents.

Contrary to the above, procedure SP-501, Halon System Functional Test, was found to be inadequate to verify operability of the cable spreading room Halon system, in that the procedure did not contain adequate instructions for use of the test equipment and data evaluation as delineated in the test equipment technical manual.

This is a Severity Level IV Violation (Supplement I.D.1).

- B. Technical Specification 6.11 requires procedures prepared to meet the requirements of 10 CFR Part 20 to be adhered to for all activities involving personnel radiation exposure.

Radiological control procedure RP-101, Radiation Protection Manual, paragraph 4.8, Use of Protective Clothing, requires protective clothing to be worn as specified on the Radiation Work Permit (RWP) when in a Radiation Control Area (RCA).

Radiological control procedure RP-106, Radiation Work Permit Procedure, paragraph 5.8, RWP Procedural Cycle, requires personnel to log their name and TLD number on the "RWP Authorized Personnel Radiation Exposure Log" to assist in radiation exposure control.

Contrary to the above, at approximately 0700 hours on March 30, a person was observed within a RCA without all the protective clothing on the RWP. In addition, this same person was not logged on the "RWP Authorized Personnel Radiation Exposure Log".

This is a Severity Level V Violation (Supplement IV.E.2)

- C. Technical Specification 3.7.1.2 requires both the steam driven and motor driven emergency feedwater pumps to be operable when in operational modes 1, 2, or 3.

Technical Specification 4.7.1.2 requires surveillance to be performed on the steam driven emergency feedwater pump within 24 hours after entering Mode 3.

Surveillance Procedure SP-349, Emergency Feedwater System Operability Demonstration, requires in step 6.3.7 that the control switch for the steam driven emergency feedwater pump stop valve (ASV-5) be placed in the "Auto" position as part of the pump's return to standby status.

Contrary to the above, at 0929 hours on April 17, with the plant in Mode 3 operations and following completion of surveillance procedure SP-349 at approximately 0830 hours, the steam driven emergency feedwater pump (EFP-2) was not operable in that the turbine stop valve (ASV-5) was not in the "Auto" position.

This is a Severity Level V Violation (Supplement I.E.).

Pursuant to the provisions of 10 CFR 2.201, you are hereby required to submit to this office within twenty-five days of the date of this Notice, a written statement or explanation in reply, including: (1) admission or denial of the alleged violations; (2) the reasons for the violations if admitted; (3) the corrective steps which have been taken and the results achieved; (4) corrective steps which will be taken to avoid further violations; and (5) the date when full compliance will be achieved. Under the authority of Section 182 of the Atomic Energy Act of 1954, as amended, this response shall be submitted under oath or affirmation.

Date: MAY 15 1981