

June 23, 1981

Docket No. 50-219
LS05-81-06-080



Mr. I. R. Finfrock, Jr.
Vice President - Jersey Central
Power & Light Company
Post Office Box 388
Forked River, New Jersey 08731

Dear Mr. Finfrock:

SUBJECT: OYSTER CREEK - LIST AND STATUS OF SYSTEMATIC EVALUATION PROGRAM,
PHASE II, GENERIC TOPICS

By letter dated May 7, 1981 (Letter to all SEP Licensees from G.C. Lainas), we sent you the list of generic topics that will be addressed in the Systematic Evaluation Program integrated assessment (see Enclosure 3 of the letter).

The letter also stated that you would be informed by separate correspondence of the status of the generic topics and what further action, if any, is needed. Enclosure 1 of this letter presents the status of these generic topics and supersedes Enclosure 3 of the May 7, 1981 letter. The status code is identical to that used in NUREG-0485 and, for convenience, is included herewith as Enclosure 2. Topics in status code 2 require a response from the licensee in order for topic review to continue. We need your response within three weeks of receipt of this letter in order to meet our agreed upon schedules.

Sincerely,

Dennis M. Crutchfield, Chief
Operating Reactors Branch No. 5
Division of Licensing

Enclosure:
As stated

cc w/enclosure:
See next page

POOR ORIGINAL

8106290 325
P

OFFICE	SEP B	SEP B	SEP B	ORB#5	ORB#5	ADPSA:CL
SURNAME	TMichaels	CBerlinger	WRussell	JLombardo	DCrutchfield	GLainas
DATE	6/15/81	6/15/81	6/15/81	6/19/81	6/24/81	6/24/81



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555
June 23, 1981

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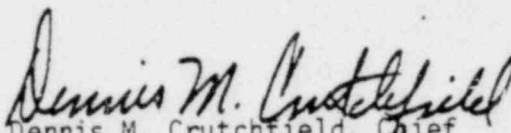
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ENCLOSURE 1

OYSTER CREEK

GENERIC TASK STATUS

TOPIC

STATUS

III-4.B - Turbine Missiles

Status 0

This topic is not generic for Millstone 1. It is being reviewed in the SEP on a plant specific basis.

III-7.A - Inservice Inspection -
Containments

Deleted - 11/16/79

III-8.A - Loose Parts Monitoring
and Core Barrel Vibrat-
ion Monitoring

Loose Parts Monitoring

Status 0

(NRR Generic Activity No. B-60 - Loose Parts Monitoring Systems. Implementation position has been developed in Revision 1 to Regulatory Guide 1.133 "Loose-Part Detection Program for the Primary System of Light-Water-Cooled Reactors." Letters will be sent to licensees requesting that a review be conducted and that a loose-parts detection program be prepared and available for inspection 6 months after the effective issuance date of the guide. Rev. 1 was issued May 28, 1981.)

Core Barrel Vibration Monitoring

Status 0

(NRR Generic Activity No. C-12 - Primary System Vibration Assessment. Implementation position has not been developed.)

I-3 - BWR Jet Pumps
Operating Indications

Deleted - 11/16/79

TOPIC

STATUS

V-1 - Compliance with Codes
and Standards

Inservice Inspection

Status 3 - 02/05/81

(Multi-Plant issue A-01 - Inservice In-
spection)

Inservice Testing

Status 2 - Valves - 09/03/80

Status 2 - Pumps - 09/03/80

(Multi-Plant issue A-14 - Inservice
Testing)

V-7 - Reactor Coolant Pump
Overspeed

Deleted - 11/16/79

VI-6 - Containment Leak Testing

Status 4 - 06/04/81

(Multi-plant issue A-04 - Appendix J Leak
Testing)

VI-7.A.2 - Upper Plenum Injection

Deleted - 11/16/79

VI-7.A.4 - Core Spray Nozzle
Effectiveness

Status 0

(Multi-Plant issue D-12 - Non-Jet Pump
BWR Core Spray Performance)

VI-7.D - Long Term Cooling
Passive Failures

Complete - 08/17/78

(Multi-Plant issue B-11, Flood of Equip-
ment Important to Safety, is not applic-
able to this topic.)

VII-1.B - Trip Uncertainty and
Setpoint Analysis Review

Complete - 08/17/78

VII-6 - Frequency Decay

Status 4 - 04/21/80

(NRR Generic Activity No. A-35. Adequacy
of offsite Power Systems. An analysis
of frequency decay has been performed un-
der subtask 8 of Task A-35. An SER will
be prepared that states there is no fre-
quency decay condition of concern at this
plant.)

VIII-1.A - Degraded Grid Voltage

Status 3 - 08/11/80

(Multi-Plant issue B-23 - Degraded Grid
Voltage)

ENCLOSURE 1

<u>TOPIC</u>	<u>STATUS</u>
IX-6 - Fire Protection	Status 7 - 03/03/78 and 02/13/81 (Fire Protection SER) Status 3 - 03/20/81 - (Safe Shutdown Capability - Appendix R reviews) (Multi-Plant Issue B-02 Fire Protection)
XI-1 - Appendix I	Status 2 - 10/24/79 (Multi-Plant Issue A-02 - Appendix I - Alara)
XI-2 - Radiological Monitoring Systems	Status 0 (NRR Generic Activity No. 8-67 - Effluent and Process Monitoring. Implementation Position has not developed for operating plants. A new Appendix A to Standard Review Plan 11.5 has been developed, which prescribes design and quality assurance criteria.)
. XIII-2 - Safeguards/Industrial Security	Complete - 09/14/79 and 09/25/80
XVII - Operational QA Program	Complete - 08/17/78

ENCLOSURE 2

KEY FOR STATUS CODES*

- | | |
|--|---|
| 0 - TOPIC NOT
STARTED | 3 - QUESTION RESPONSE
RECEIVED BY NRC |
| 1 - QUESTIONS RECEIVED
FROM REVIEW BRANCH | 4 - DRAFT SE RECEIVED
FROM REVIEW BRANCH |
| 2 - QUESTIONS SENT TO
UTILITY | 5 - DRAFT SE SENT TO
UTILITY |
| | 6 - SE RESPONSE RECEIVED
BY NRC |
| | 7 - TOPIC COMPLETE |

*Date following a status code in Enclosure 1 refers to either a letter from the NRC to Licensee or from the Licensee to NRC.