

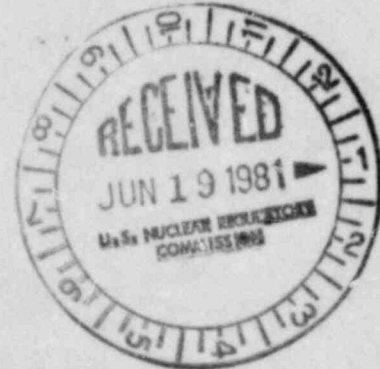


UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555
JUN 18 1981

TERA

Docket No. 50-317

Mr. A. E. Lundvall, Jr.
Vice President - Supply
Baltimore Gas & Electric Company
P. O. Box 1475
Baltimore, Maryland 21203



Dear Mr. Lundvall:

We are currently reviewing your February 4, 1981 submittal of "Calvert Cliffs Unit No. 1 Nuclear Plant Reactor Pressure Vessel Surveillance Program: Capsule 263". We find that additional information as delineated in the enclosure is necessary to complete our review.

Please provide the requested additional information at your earliest convenience.

Sincerely,

for Charles M. Trammell
Robert A. Clark, Chief
Operating Reactors Branch #3
Division of Licensing

Enclosure: As stated

cc: See next page

8106260236

Baltimore Gas and Electric Company

cc:

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Combustion Engineering, Inc.
Attn: Mr. P. W. Kruse, Manager
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CALVERT CLIFFS NUCLEAR POWER PLANT UNIT 1
REVIEW OF PRESSURE TEMPERATURE LIMITS

REQUEST FOR ADDITIONAL INFORMATION

BALTIMORE GAS AND ELECTRIC
DOCKET NO. 50-317

1. Provide a map of the end-of-life isofluence at the internal surface of the beltline region of the reactor vessel, showing a) the location of the longitudinal and girth welds in relation to the core configuration, and b) the location of the materials surveillance capsules.
2. Identify the computer code used for calculating the fluence.
3. In the event that other materials surveillance capsules have been removed from the reactor, a) compare the predicted with the actual fluence values as determined by dosimetry, b) identify the published surveillance reports, and c) state when the next surveillance capsule is scheduled to be removed from the reactor.
4. Provide a transverse operational cross-section of the reactor vessel in the beltline region showing the reactor internal components and the location of the surveillance capsules. Indicate the dimensions of the materials interposed at full power density between the core and the walls of the reactor vessel.
5. Explain why the capsule at 263° was pulled and tested instead of the one at 97°. What is the future planned removal sequence? Justify briefly.