



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

MAINE YANKEE ATOMIC POWER COMPANY

DOCKET NO. 50-309

MAINE YANKEE ATOMIC POWER STATION

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 57
License No. DPR-36

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Maine Yankee Atomic Power Company (the licensee) dated May 22, 1981 complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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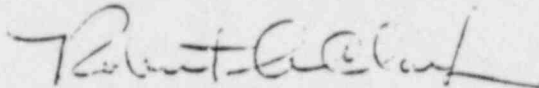
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.B(6)(b) of Facility Operating License No. DPR-36 is hereby amended to read as follows:

(b) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 57, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment was effective on May 22, 1981.

FOR THE NUCLEAR REGULATORY COMMISSION



Robert A. Clark, Chief
Operating Reactors Branch #3
Division of Licensing

Attachments:
Changes to the Technical
Specifications

Date of Issuance: June 12, 1981

ATTACHMENT TO LICENSE AMENDMENT NO. 57
TO FACILITY OPERATING LICENSE NO. DPR-36
DOCKET NO. 50-309

Revise Appendix A as follows:

Remove Pages

3.17-3
3.17-6

Insert Pages

3.17-3
3.17-6

3. a. During normal conditions of plant operation, radioactive gaseous waste from the hydrogenated waste gas system shall be provided a minimum average holdup of 60 days except for low radioactivity gaseous waste resulting from purge and fill operations associated with refueling and reactor startup.
- b. Holdup time less than that specified in B.3.a above shall be covered in the special effluent report to be included in the semi-annual report required by Section 5.7.B.(1)(a) of these specifications.
- c. The maximum activity to be contained in one gas decay tank shall not exceed 88,400 curies of Xe-133 equivalent.
4. During the first indication of primary-to-secondary leakage, concurrent with sufficient fuel defects, a determination of the iodine partition factor for the blowdown tank shall be made.
5. During power operation, the condenser air ejector discharge shall be continuously monitored for gross radiogas activity. Whenever this monitor is inoperable, grab samples shall be taken from the air ejector discharge and analyzed for gross radiogas activity daily.
6. Gases discharged through the stack shall be continuously monitored for gross noble gas and particulate activity. Whenever either of these monitors is inoperable, appropriate grab samples shall be taken and analyzed daily.
7. Purging of the reactor building shall be governed by the following conditions:
 - a. Reactor building purge shall be filtered through the high efficiency particulate air filters and charcoal absorbers whenever the concentration of iodine and particulate isotopes exceeds the occupational MPC inside the reactor building.
 - b. Reactor building purge shall be filtered through the high efficiency particulate air filters and charcoal absorbers whenever irradiated fuel is being handled or any object is being handled over irradiated fuel in the reactor building.

Exception: The requirements of Par. 7b above shall not apply from May 22, 1981 until either completion of the LPSI check valve installation ordered April 20, 1981 or completion of operations involving irradiated fuel handling or handling of any object over irradiated fuel during the cycle 5/6 refueling, whichever is earlier. When refueling operations are conducted under this exception, the containment purge valves shall be trippable manually and automatically.

The quantity and isotopic proportions of radioactive gases released into the reactor coolant system is dependent upon several factors including fuel leakage, burnup and power level. Changes in power level will affect gaseous generation rates temporarily. Gases are released from the reactor coolant to the gaseous waste system during degassifier treatment of the letdown and leakage water and also during venting of the system. This venting may occasionally be performed to degas the system and so control plant chemistry and/or reduce coolant radioactive gas concentrations to an acceptable value for the protection of plant personnel.

Gaseous waste holdup and decay occurs while it is retained in the reactor coolant system and in the surge drum of the gaseous treatment system. The gaseous waste holdup drums are of sufficient capacity to provide an additional average retention period of 60 days during normal operating conditions.

The low as practicable gaseous release objectives expressed in this Specification are based on the guidelines contained in the proposed Appendix I of 10 CFR 50. Since these guidelines have not been adopted as yet, the release objectives of this Specification will be reviewed at the time Appendix I becomes a regulation to assure that this Specification is based upon the guidelines contained therein.

Basis: The exception to Par. 3.17.B.7.b allows installation of additional LPSI check valves reducing the probability of event V LOCA to proceed without extending unnecessarily the cycle 5/6 refueling outage.

Due to the extended time period between reactor shutdown and the present there has been considerable decay of the radioactive fission products in the fuel. An analysis of a refueling accident inside containment shows the doses would be appropriately within the guidelines of 10 CFR 100 (specifically in this case on the order of 100 rem thyroid) taking no credit for either filtration or containment isolation.