



UNITED STATES
 NUCLEAR REGULATORY COMMISSION
 REGION IV
 611 RYAN PLAZA DRIVE, SUITE 1000
 ARLINGTON, TEXAS 76012

March 24, 1981

MEMORANDUM FOR: Those Listed Below
 FROM: G. L. Madsen, Chief, Reactor Projects Branch, RIV
 SUBJECT: IE INFORMATION NOTICE NO. 81-08

Subject IE Information Notice has been sent to the following listed licensees. A copy of the letter and the IE Information Notice sent to each licensee is attached for your information.

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| Arkansas Power & Light Company
ANO-1 & 2 (50-313; 50-368) | Gulf States Utilities
River Bend (50-458; 50-459) |
| Nebraska Public Power District
Cooper Station (50-298) | Houston Lighting & Power Company
South Texas (50-498; 50-499) |
| Omaha Public Power District
Ft. Calhoun Station (50-285) | Kansas Gas & Electric Company
Wolf Creek (STN 50-482) |
| Public Service Company of
Colorado
Fort St. Vrain (50-267) | Louisiana Power & Light Company
Waterford-3 (50-382) |
| | Texas Utilities Generating Company
Comanche Peak (50-445; 50-446) |

G. L. Madsen
 G. L. Madsen, Chief,
 Reactor Projects Branch

Attachments:
 As stated

DISTRIBUTION:
 IE/DMS
 MPA/LOEB
 IE/RPRIB
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 IE/SRSI
 IE/EP
 ADM/OMB



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Accession No.:
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IN 81-08

UNITED STATES
NUCLEAR REGULATORY COMMISSION
OFFICE OF INSPECTION AND ENFORCEMENT
WASHINGTON, D. C. 20555

IE Information Notice No. 81-08
March 20, 1981
Page 1 of 1

REPETITIVE FAILURES OF LIMITORQUE OPERATOR
SMB-4 MOTOR-TO-SHAFT KEY

Description of Circumstances:

During normal surveillance testing of motor-operated valves at the Cooper Nuclear Station, it was discovered that the RHR "B" loop suppression pool cooling inboard throttle valve would not operate. The valve did not operate because the key between the motor pinion gear and the motor shaft had failed. The operator for the valve has been identified as being Limitorque Operator Model SMB-4. Analysis of the failed key revealed that a standard type 1010 steel key had been installed instead of the required special type 4140 steel key.

Further review of the problem established that similar failures with motor-to-shaft keys had also taken place at the Pilgrim, Hatch, and Fitzpatrick nuclear power facilities.

Limitorque Company has performed an evaluation of the problem, and they have concluded that the problem is confined to operators having a torque rating in excess of 100 ft/lb. The standard type 1010 steel possesses adequate strength to accommodate motor operators having a torque rating that is less than 100 ft/lb.

Recommended Action for Holders of Operating Licenses and Construction Permits:

It is recommended that licensees having Limitorque Operator Model SMB-4 installed in their facilities verify that the special type 4140 steel keys have been used. This can be easily accomplished by performing a hardness check of the motor-to-shaft keys during the next scheduled surveillance testing.

No written response to this information notice is required. If you need additional information with regard to this subject, please contact the Director of the appropriate NRC Regional Office.

IE Information Notice No. 01-08
March 20, 1981

LISTING OF RECENTLY ISSUED
IE INFORMATION NOTICES

Information Notice No.	Subject	Date Issued	Issued To
80-45	Potential Failure of BWR Backup Manual Scram Capability	12/17/80	All pressurized water reactor facilities with an Operating License (OL) Construction Permit (CP)
81-01	Possible Failures of General Electric Type HFA Relays	1/16/81	All power reactor facilities with an Operating License (OL) Construction Permit (CP)
81-02	Transportation of Radiography Devices	1/23/81	All Radiography licensees
81-03	Checklist for Licensees Making Notifications of Significant Events in Accordance with 10 CFR Part 50.72	2/12/81	All power reactor facilities with an Operating License (OL) and Near-term Applicants
81-04	Cracking in Main Steam Lines	2/27/81	All power reactor facilities with an Operating License (OL) or Construction Permit (CP)
81-06	Failure of ITE Model K-600 Circuit Breaker	3/11/81	All power reactor facilities with an Operating License (OL) or Construction Permit (CP)
81-05	Degraded DC System at Palisades	3/13/81	All power reactor facilities with an Operating License (OL) or Construction Permit (CP)
81-07	Potential Problem with Water-Soluble Purge Dam Materials Used During Inert Gas Welding	3/16/81	All power reactor facilities with an Operating License (OL) or Construction Permit (CP)

Enclosure