



Metropolitan Edison Company  
Post Office Box 480  
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Writer's Direct Dial Number

June 12, 1981  
L1L 180

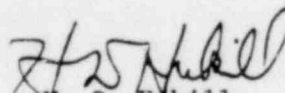
Office of Nuclear Reactor Regulation  
Attn: John F. Stolz, Chief  
Operating Reactors Branch No. 4  
U. S. Nuclear Regulatory Commission  
Washington, D.C. 20555

Dear Sir:

Three Mile Island Nuclear Station, Unit 1 (TMI-1)  
Operating License No. DPR-50  
Docket No. 50-289  
Environmental Qualification for Small Break LOCA

The purpose of this letter is to confirm information transmitted by telephone June 5 to June 11, 1981 for clarification of our submittal dated May 18, 1981 (L1L 161).

Sincerely,

  
H. D. Hickill  
Director, TMI-1

HDH:CWS:lma

Enclosure

cc: R. Jacobs  
D. DiIanni



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Question: What radiation levels are expected in the vicinity of Valves DH-V-4A/B and DH-V-5A/B during normal operation and during DH System operation?

Answer: General area dose rates in the vicinity of DH-V-4A/B and DH-V-5A/B are less than 100mr/hour\* (and usually much less than 100mr/hr) during either normal operation or DH System operation. This is based on routine radiation surveys taken since TMI-1 began operation.

Question: Will GPU review the results of the Westinghouse tests on Reliance motors used on Limitorque actuators? Will GPU advise the NRC if these test results are applicable to TMI-1 and the effects, if any?

Answer: All information that GPU has reviewed to date indicates that the Reliance motors used in TMI-1 Limitorque actuators are qualified as specified in our 79-01B submittal.

GPU will review and comment to the NRC on the Westinghouse reports after the NRC makes the reports available to GPU.

Question: Do the failures on Foxboro transmitters, described in test report T3-1068, affect the qualification of the TMI-1 transmitters? Are these the same type of units?

Answer: All units in this report continued to function up to  $7.6 \times 10^7$  R. This is orders of magnitude above the SB LOCA radiation for TMI-1. The TMI-1 transmitters are of the same type as those tested.

Question: When will the B & W Report "Evaluation of Aging of Class IE Controls and Instrumentation in B & W 177FA Scope of Supply" be completed?

Answer: B & W has stated that the report should be published by July 15, 1981. The report is being sponsored by the B & W Owners Group and its publication is under control of B & W and the Owners Group.

Question: Will all components that have exceeded their qualified life expectancy be replaced before Restart? Will a program to replace components, as needed, be in place by Restart?

Answer: Yes. The only such components identified to date are neoprene cover seal gaskets. In addition, a procedure will be implemented by criticality to replace components as needed.

\*100 mr/hr may be briefly exceeded for up to 24 hours following a shutdown involving a large crud burst.