



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555
June 12, 1981



Docket No. 50-29
LS05-81-06-041

Mr. James A. Kay
Senior Engineer - Licensing
Yankee Atomic Electric Company
1671 Worcester Road
Framingham, Mass. 01701

Dear Mr. Kay:

SUBJECT: YANKEE ROWF - LIST AND STATUS OF SYSTEMATIC EVALUATION PROGRAM,
PHASE II, GENERIC TOPICS

By letter dated May 7, 1981 (Letter to all SEP Licensees from G.C. Lainas), we sent you the list of generic topics that will be addressed in the Systematic Evaluation Program integrated assessment (see Enclosure 3 of the letter).

The letter also stated that you would be informed by separate correspondence of the status of the generic topics and what further action, if any, is needed. Enclosure 1 of this letter presents the status of these generic topics and supersedes Enclosure 3 of the May 7, 1981 letter. The status code is identical to that used in NUREG-0485 and, for convenience, is included herewith as Enclosure 2. There are no actions required by you at this time regarding the generic topics in Enclosure 1.

Sincerely,

Dennis M. Crutchfield
Dennis M. Crutchfield, Chief
Operating Reactors Branch No. 5
Division of Licensing

Enclosure:
As stated

cc w/enclosure:
See next page

*Add: Gary
Staley-ley*

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Mr. James A. Kay

cc

Mr. James E. Tribble, President
Yankee Atomic Electric Company
25 Research Drive
Westborough, Massachusetts 01581

Greenfield Community College
1 College Drive
Greenfield, Massachusetts 01301

Chairman
Board of Selectmen
Town of Rowe
Rowe, Massachusetts 01367

Energy Facilities Siting Council
14th Floor
One Ashburton Place
Boston, Massachusetts 02108

Director, Criteria and Standards
Division
Office of Radiation Programs
(ANR-460)
U. S. Environmental Protection
Agency
Washington, D. C. 20460

U. S. Environmental Protection
Agency
Region I Office
ATTN: EIS COORDINATOR
JFK Federal Building
Boston, Massachusetts 02203

Resident Inspector
Yankee Rowe Nuclear Power Station
c/o U.S. NRC
Post Office Box 28
Monroe Bridge, Massachusetts 01350

ENCLOSURE 1

YANKEE ROWE

GENERIC TASK STATUS

<u>TOPIC</u>	<u>STATUS</u>
III-4.B - Turbine Missiles	Status 3 - 02/17/81 (Multi-Plant issue B-46 - Analysis of Turbine Disc Cracks)
III-7.A - Inservice Inspection - Containments	Deleted - 11/16/79
III-8.A - Loose Parts Monitoring and Core Barrel Vibration Monitoring	<u>Loose Parts Monitoring</u> Status 0 (NRR Generic Activity No. B-60 - Loose Parts Monitoring Systems. Implementation position has been developed in Revision 1 to Regulatory Guide 1.133 "Loose-Part Detection Program for the Primary System of Light-Water-Cooled Reactors." Letters will be sent to licensees requesting that a review be conducted and that a loose-parts detection program be prepared and available for inspection 6 months after the effective issuance date of the guide. Rev. 1 was issued May 28, 1981.)
	<u>Core Barrel Vibration Monitoring</u> Status 0 (NRR Generic Activity No. C-12 - Primary System Vibration Assessment. Implementation position has not been developed.)
IV-3 - BWR Jet Pumps Operating Indications	Deleted - 11/16/79

<u>TOPIC</u>	<u>STATUS</u>
V-1 - Compliance with Codes and Standards	<u>Inservice Inspection</u> Status 3 - 06/26/79 (Multi-Plant issue A-01 - Inservice Inspection)
	<u>Inservice Testing</u> Status 4 - Valves - 08/04/80 Status 3 - Pumps - 06/29/79 (Multi-Plant issue A-14 - Inservice Testing)
V-7 - Reactor Coolant Pump Overspeed	Status 0 (NRR Generic Activity No. B-68 - Pump Overspeed during a LOCA. Implementation position has not been developed.)
VI-6 - Containment Leak Testing	Status 3 - 04/27/81 (Multi-plant issue A-04 - Appendix J Leak Testing)
VI-7.A.2 - Upper Plenum Injection	Deleted - 11/16/79
VI-7.A.4 - Core Spray Nozzle Effectiveness	Deleted - 11/16/79
VI-7.D - Long Term Cooling Passive Failures	Complete - 08/17/78 (Multi-Plant issue B-11, Flood of Equipment Important to Safety, is not applicable to this topic.)
VII-1.B - Trip Uncertainty and Setpoint Analysis Review	Complete - 08/17/78
VII-6 - Frequency Decay	Status 4 - 04/21/80 (NRR Generic Activity No. A-35. Adequacy of offsite Power Systems. An analysis of frequency decay has been performed under subtask 8 of Task A-35. An SER will be prepared that states there is no frequency decay condition of concern at this plant.)
VIII-1.A - Degraded Grid Voltage	Status 3 - 05/01/81 (Multi-Plant issue B-23 - Degraded Grid Voltage)

<u>TOPIC</u>	<u>STATUS</u>
IX-6 - Fire Protection	Status 7 - 03/15/78 (Fire Protection SER) Status 3 - 03/19/81 - (Safe Shutdown Capability - Appendix R reviews) (Multi-Plant Issue B-02 Fire Protection)
XI-1 - Appendix I	Status 3 - 12/17/79 (Multi-Plant Issue A-02 - Appendix I - Alara)
XI-2 - Radiological Monitoring Systems	Status 0 (NRR Generic Activity No. B-67 - Effluent and Process Monitoring. Implementation Position has not been developed for operating plants. A new Appendix A to Standard Review Plan 11.5 has been developed, which prescribes design and quality assurance criteria.)
XIII-2 - Safeguards/Industrial Security	Complete - 03/11/81
XVII - Operational QA Program	Complete - 08/17/78

ENCLOSURE 2

KEY FOR STATUS CODES*

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|--|---|
| 0 - TOPIC NOT
STARTED | 3 - QUESTION RESPONSE
RECEIVED BY NRC |
| 1 - QUESTIONS RECEIVED
FROM REVIEW BRANCH | 4 - DRAFT SE RECEIVED
FROM REVIEW BRANCH |
| 2 - QUESTIONS SENT TO
UTILITY | 5 - DRAFT SE SENT TO
UTILITY |
| | 6 - SE RESPONSE RECEIVED
BY NRC |
| | 7 - TOPIC COMPLETE |

*Date following a status code in Enclosure 1 refers to either a letter from the NRC to Licensee or from the Licensee to NRC.

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