



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

COMMONWEALTH EDISON COMPANY

DOCKET NO. 50-237

DRESDEN STATION, UNIT NO. 2

AMENDMENT TO PROVISIONAL OPERATING LICENSE

Amendment No. 60
License No. DPR-19

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Commonwealth Edison Company (the licensee) dated March 27, 1981, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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POOR QUALITY PAGES

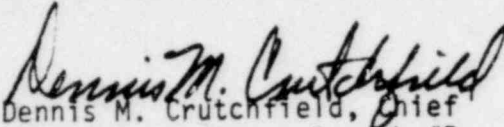
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 3.B of Provisional Operating License No. DPR-19 is hereby amended to read as follows:

3.B Technical Specifications

The Technical Specifications contained in Appendix A as revised through Amendment No. 6Q, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION


Dennis M. Crutchfield, Chief
Operating Reactors Branch #5
Division of Licensing

Date of Issuance: May 6, 1981

ATTACHMENT TO LICENSE AMENDMENT NO. 60
PROVISIONAL OPERATING LICENSE NO. DPR-19
DOCKET NO. 50-237

Replace page 91b, 91d, and 91e with the enclosed page 91b and add the following new pages: 91 d-1, 91 d-2, 91 e-1 and 91 e-2

3.6 LIMITING CONDITION FOR OPERATION

I. Shock Suppressors (Snubbers)

- i. During all modes of operation except cold shutdown and refuel, all safety-related snubbers listed in Tables 3.6.1a and b shall be operable except as noted in Specification 3.6.I.2 through 3.6.I.4.
2. From and after the time that a snubber is determined to be inoperable, continued reactor operation is permissible only during the succeeding 72 hours unless the snubber is sooner made operable or replaced. Torus ring header snubbers may be inoperable in groups of up to three (3) pairs until September 1, 1980, to facilitate the installation of Mark I Torus Support Modifications.
3. If the requirements of 3.6.I.1 and 3.6.I.2 can not be met, an orderly shutdown shall be initiated and the reactor shall be in cold shutdown or refuel condition within 36 hours.
4. If a snubber is determined to be inoperable while the reactor is in the cold shutdown or refuel mode, the snubber shall be made operable or replaced prior to reactor startup. This requirement does not apply to torus ring header snubbers for the period identified in paragraph 3.6.I.2 above.
5. Snubbers may be added to safety related systems without prior license amendment to Tables 3.6.1a and b provided that a revision to Tables 3.6.1a and b is included with the next license amendment request.

4.6 SURVEILLANCE REQUIREMENT

I. Shock Suppressors (Snubbers)

The following surveillance requirements apply to all hydraulic snubbers listed in Table 3.6.1a.

1. All hydraulic snubbers whose seal material has been demonstrated by operating experience, lab testing or analysis to be compatible with the operating environment shall be visually inspected. This inspection shall include, but not necessarily be limited to inspection of the hydraulic fluid reservoir, fluid connections, and linkage connection to the piping and anchor to verify snubber operability in accordance with the following schedule:

<u>No. of Snubbers Found Inoperable During Inspection Interval</u>	<u>Next Required Inspection Interval</u>
0	18 months \pm 25%
1	12 months \pm 25%
2	6 months \pm 25%
3, 4	124 days \pm 25%
5, 6, 7	62 days \pm 25%
\geq 8	31 days \pm 25%

The required inspection interval shall not be lengthened more than one step at a time.

TABLE 3.6.1a

SAFETY RELATED HYDRAULIC SNUBBERS

SNUBBER NO.	LOCATION	ELEVATION	AZIMUTH	SNUBBER IN HIGH RADIATION AREA DURING SHUTDOWN	SNUBBERS INACCESSIBLE DURING NORMAL OPERATION	SNUBBERS ACCESSIBLE DURING NORMAL OPERATION
2	Torus Ring Header 1501-24"	483'	83°			X
3	Torus Ring Header 1501-24"	483'	74°			X
4	Torus Ring Header 1501-24"	483'	38°			X
5	Torus Ring Header 1501-24"	483'	29°			X
7	Torus Ring Header 1501-24"	483'	331°			X
8	Torus Ring Header 1501-24"	483'	322°			X
9	Torus Ring Header 1501-24"	483'	286°			X
10	Torus Ring Header 1501-24"	483'	277°			X
12	Torus Ring Header 1501-24"	483'	218°			X
13	Torus Ring Header 1501-24"	483'	209°			X
15	Torus Ring Header 1501-24"	483'	151°			X
16	Torus Ring Header 1501-24"	483'	142°			X

*Modifications to this table due to changes in high radiation should be submitted to the NRC as part of the next license amendment request.

TABLE 3.6.1a

SAFETY RELATED HYDRAULIC SNUBBERS

SNUBBER NO.	LOCATION	ELEVATION	AZIMUTH	SNUBBER IN HIGH RADIATION AREA DURING SHUTDOWN	SNUBBERS INACCESSIBLE DURING NORMAL OPERATION	SNUBBERS ACCESSIBLE DURING NORMAL OPERATION
	Isolation Condenser Pipeway Room:					
1	Isolation Condenser Line 1303-12"	558'	180°	X		X
2	Isolation Condenser Line 1303-12"	568'	180°	X		X
3	Isolation Condenser Line 1302-14"	580'	195°	X		X

Modifications to this table due to changes in high radiation should be submitted to the NRC as part of the next license amendment request.

TABLE 3.6.1b

SAFETY RELATED MECHANICAL SNUBBERS

SNUBBER NO.	LOCATION	ELEVATION	AZIMUTH	SNUBBER IN HIGH RADIATION AREA DURING SHUTDOWN	SNUBBERS INACCESSIBLE DURING NORMAL OPERATION	SNUBBERS ACCESSIBLE DURING NORMAL OPERATION
1	Drywell Recirc. Motor 2B-202	524'	328°	X	X	
2	Drywell Recirc. Motor 2B-202	524'	302°	X	X	
3	Drywell Recirc. Motor 2B-202	524'	315°	X	X	
4	Drywell Recirc. Motor 2A-202	524'	148°	X	X	
5	Drywell Recirc. Motor 2A-202	524'	122°	X	X	
6	Drywell Recirc. Motor 2A-202	524'	135°	X	X	
7	Drywell Recirc. Pump 2B-202	512'	326°	X	X	
8	Drywell Recirc. Pump 2B-202	512'	304°	X	X	
9	Drywell Recirc. Pump 2B-202	517'	315°	X	X	
10	Drywell Recirc. Pump 2A-202	512'	124°	X	X	
11	Drywell Recirc. Pump 2A-202	512'	146°	X	X	
12	Drywell Recirc. Pump 2A-202	507'	135°	X	X	
13-16	Removed					
17	Drywell Recirc Header 201B-22"	533'6"	195°	X	X	
18-20	Removed					
21	Drywell Recirc Header 201A-22"	533'6"	22°	X	X	
22-23	Removed					
24	Drywell Feedwater Line 3204D-12"	538'	108°	X	X	
25-29	Removed					
30	Drywell Core Spray Line 1403-10"	575'	336°	X	X	
31	Drywell Core Spray Line 1404-10"	562'	231°	X	X	
32	Drywell Target Rock Valve 203-3A	542'6"	16°	X	X	
33	Drywell Target Rock Valve 203-3A	542'4"	31°	X	X	
34	Drywell Target Rock Valve 203-3A	540'0"	19°	X	X	
35	Drywell Target Rock Valve 203-3A	540'3"	34°	X	X	
36	Drywell Recirc. Line 201B-28"	518'	270°	X	X	

Modifications to this table due to changes in high radiation should be submitted to the NRC as part of the next license amendment request.

TABLE 3.6.1b

SAFETY RELATED MECHANICAL SNUBBERS

SNUBBER NO.	LOCATION	ELEVATION	AZIMUTH	SNUBBER IN HIGH RADIATION AREA DURING SHUTDOWN	SNUBBERS INACCESSIBLE DURING NORMAL OPERATION	SNUBBERS ACCESSIBLE DURING NORMAL OPERATION
37	Drywell Recirc. Line 201A-28"	518'	90°	X	X	
38	Drywell Shutdown Cooling Line 1001A-16"	523'	0°	X	X	
39	Drywell Rx Water Cleanup Line 1201-8"	533'	316°	X	X	
40	Drywell Rx Water Cleanup Line 1201-8"	533'	301°	X	X	

*Modifications to this table due to changes in high radiation should be submitted to the NRC as part of the next license amendment request.