

Detroit
Edison

ENRICO FERMI UNIT 2 PROJECT
ENGINEERING

June 8, 1981
EF2-53491



Mr. L. L. Kintner
Division of Project Management
Office Of Nuclear Regulation
U. S. Nuclear Regulatory Commission
Washington, D. C. 20555

Dear Mr. Kintner:

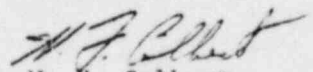
Reference: Enrico Fermi Atomic Power Plant, Unit 2
NRC Docket No. 50-341

Subject: Amended FSAR Appendix H, Section H.II.D.1.3 Response,
Safety/Relief Valve Test Requirements

To comply with the Mechanical Engineering Branch position on NUREG - 0737, II.D.1 for SR/V test requirements, Edison has revised its position stated in Appendix H, Section H.II.D.1.3. A draft of the revised FSAR section is included in the attachment to this letter.

Should you have any additional questions, please contact Mr. D. F. Lehnert (313-649-7583).

Sincerely,


W. F. Colbert
Technical Director
Fermi 2 Project

DFL/slm
Attachment

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ATTACHMENT FOR EF2-53491

H.II.D.1.3 Detroit Edison Position

On September 17, 1980, the BWR Owners' Group submitted documents entitled NUREG - 0578 BWR Safety/Relief Valve Test Description and Event Evaluation for BWR Safety Relief Valve Testing Required by NUREG - 0578, 2.1.2 (Reference 1). The test program description and basis described in this submittal is applicable to Fermi 2 and is endorsed by Detroit Edison.

The results of the BWR Owners' Group evaluation indicate that there is one event and single-failure combination that would lead to the discharge of liquid from the safety/relief valves. This event and single-failure combination lead to the alternate shutdown mode of operation that uses the safety/relief valves as a return flow path for low-pressure liquid to the suppression pool. The evaluation demonstrates that all other events postulated to produce liquid or two-phase safety/relief valve flow, including events under high-pressure conditions, are either of sufficiently low probability or that consequences are concluded to be acceptable. As such, no testing is needed for these events.

The BWR Owners' Group testing program includes the testing of typical safety/relief valves for BWR/2 through BWR/6 plants to demonstrate the ability to perform satisfactorily under the condition in which low-pressure (i.e., up to 250 ± 20 psig) water passes through the valve instead of saturated steam. This corresponds to conditions expected during the alternate shutdown cooling mode, that is, the mode in which low-pressure pumps are injecting cold water into the reactor vessel and this water is vented through the safety/relief valves back to the suppression pool. A plant-specific evaluation of the test data correlated the generic program test conditions to the alternate shutdown cooling mode conditions for Fermi 2.

The preliminary safety/relief valve (S/RV) operability test results demonstrating that the Fermi-2 S/RV satisfies the acceptance criteria for operability is to be submitted to the NRC by July 1, 1981. The final test report is to be submitted to the NRC by October 1, 1981. The evaluation of the associated discharge piping is to be submitted by January 1, 1982. In addition, Edison will participate in development of any additional information applicable to the Fermi-2 docket with respect to S/RV operability and system functionality on a schedule consistent with that agreed to between BWR Owners Group and the NRC staff.

bcc:
(with attachments)

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