

U. S. NUCLEAR REGULATORY COMMISSION  
OFFICE OF INSPECTION AND ENFORCEMENT  
REGION IV

IE Inspection Report: 50-267/81-08

License No. DPR-34

Docket: 50-267

Licensee: Public Service Company of Colorado  
P. O. Box 840  
Denver, Colorado 80201

Facility Name: Fort St. Vrain Nuclear Generation Station

Inspection at: Fort St. Vrain Site, Platteville, Colorado

Inspection Conducted: April 1-30, 1981

Inspectors: T. F. Westerman for 5-21-81  
M. W. Dickerson, Senior Resident Reactor Inspector Date

T. F. Westerman for 5-21-81  
G. L. Plumlee III, Resident Reactor Inspector Date

Approved By: T. F. Westerman 5-21-81  
T. F. Westerman, Chief, Reactor Project Section Date  
No. 1

Inspection Summary

Inspection conducted on April 1-30, 1981 (Report: 50-267/81-08)

Areas Inspected: Routine, announced inspection of Surveillance; Maintenance; Operational Safety Verification; Plant Operations; Licensee Event Follow-up; IE Circular Follow-up; Followup on Inspector Identified Problems and Unresolved Problems; and review of periodic and special reports. The inspection involved 197 inspector-hours on site by two NRC inspectors.

Results: Within the seven areas inspected, no items of noncompliance were identified.

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1. Persons Contacted

G. Billings, Shift Supervisor  
L. Grey, QA Manager  
W. Franek, Results Supervisor  
J. Gahn, Supervisor Technical Services  
W. Franklin, Shift Supervisor  
E. Hill, Superintendent of Operation  
W. Hillyard, Administrative Services Manager  
F. Mathie, Operations Manager  
L. Singleton, Superintendent Operation QA  
J. Vandyke, Training Supervisor  
R. Wadas, Shift Supervisor  
D. Warembourg, Manager, Nuclear Production

The inspector also contacted other plant personnel including reactor operators, maintenance men, electricians, technicians and administrative personnel.

2. Licensee Action on Previous Inspection Findings

(Open) Unresolved Item (50-267/7502-3): Fission Product Probe. The licensee has scheduled the removal of the fission plateout probe for the next scheduled refueling shutdown. The anticipated date of this shutdown is May 1981.

(Open) Open Item (50-267/7607-01): FSAR Amendments. The licensee has contracted the services of a consultant, Nuclear Power Consultants, Inc., to assist in the update of the FSAR. Rewrite began in March 1981, and is scheduled for submission in mid-1982.

(Open) Unresolved Item (50-267/7607-4): Seismic and Environmental Qualifications. Two equipment items still require clarification of certifications. These items deal with CN 892 and the Cowan Actuators. All of the information relative to CN 892 has been gathered and verification of the data is necessary. The Cowan Actuators are to be disassembled for measurements during the refueling shut-down scheduled in May 1981.

Additionally, the two-inch and under Class I Piping Support Program is in progress and is expected to be completed by the end of 1982.

(Open) Open Item (50-267/7610-01): Power Ascension Program. B-Series testing above 70% power is partially complete and will be completed as the rise to 100% power is continued.

(Open) Unresolved Item (50-267/7613-01): Plant Operating Cycles. A procedure for data collection is in place and data is being collected and transmitted to the licensee's Nuclear Projects Department for their use. New allowable numbers of design transients have also been developed. Additionally, the Nuclear Projects Department has engaged General Atomic Company to develop a usage factor for each worst case location resulting from transients that are less severe than the current design transients. It is anticipated that this will be developed by February 1, 1982.

(Open) Unresolved Item (50-267/7714-01): Pulled Fuses. The licensee's Nuclear Projects Department expects to start an audit to identify each fuse block and size fuse required on July 1, 1981, and to be completed within one year.

(Open) Unresolved Item (50-267/8008-01): Radio Transmitters caused upset in Buffer-Mid-Buffer System. Test T-154, a rerun of T-148, has been completed. The results of this test have been sent to the licensee's Nuclear Projects Department for their evaluation and recommendations for a solution.

(Open) Open Item (50-267/8009-01): Responsibility for Review of Maintenance Activities for Equipment Trends. This is not yet described in a procedure but a computer program and data base are being investigated. The program is expected to be operational by July 1, 1981.

(Open) Open Item (50-267/8018-02): SR 5.10.3-M 5.4.2-A Requires Modification. Modification has been made to the surveillance procedure which included a new number. The new number is now SR 5.10.3.b-SA/5.4.2-A. A change to Procedure P-10 has also been approved and is waiting issuance.

(Open) Open Item (50-267/8025-02): Moisture Monitor Instrument Penetration Outer Covers Removed. A request has been issued to the Nuclear Projects Department for a safety analysis.

(Open) Unresolved Item (50-267/8025-03): Clearance Point Form Approvals. A request for a revision to Procedure P-2 to clarify the required approvals has been issued.

(Open) Unresolved Item (50-267/8025-05): Valves not Tagged or Labeled. Change Notice number 934 will assign valve/instrument numbers and request that tagging of the valves be completed. This is to be completed April 1, 1982.

(Open) Open Item (50-267/8025-06): DVM #M3147, No Calibration Data. This digital voltmeter has been sent out by the licensee to be calibrated. Additionally, the licensee has indicated that no other instruments of this type will be ordered with calibration from the manufacturer. Calibration will be performed by the licensee prior to its use.

(Closed) Unresolved Item (50-267/8108-02): Conflict of Valves Listed in SOP-11 and PI-11-3. The sealed valve list was revised to include V-111057 and V-11086P.

### 3. Operational Safety Verification

The NRC inspector reviewed licensee activities to ascertain that the facility is being operated safely and in conformance with regulatory requirements, and the licensee's management control system is effectively discharging its responsibilities for continued safe operation. The review was conducted by direct observation of activities, tours of the facility, interviews and discussion with licensee personnel, independent verification of safety system status and limiting conditions for operations, and review of facility records.

Included in the inspection were observation of control room activities, review of operational logs, records, and tours of accessible areas. Logs and records reviewed included:

- . Shift Supervisor Logs
- . Reactor Operator Logs
- . Equipment Operator Logs
- . Auxiliary Operator Logs
- . Technical Specification Compliance Logs
- . Operations Order Book
- . System Status Log
- . Form 1 Log (Jumper Log)
- . Plant Trouble Reports

During tours of accessible areas, particular attention was directed to the following:

- . Monitoring Instrumentation
- . Radiation Controls
- . Housekeeping
- . Fluid Leaks
- . Piping Vibrations
- . Hanger/Seismic Restraints
- . Clearance Tags
- . Fire Hazards
- . Control Room Manning
- . Annunciators

The operability of selected systems or portions of systems were verified by walkdown of the accessible portions. Observed were the Helium Storage System, Nitrogen System, and the Feedwater and Condensate System. Procedures were also reviewed and implementation observed for Liquid Waste Release No. 449 and Gaseous Effluent Release No. 528. The releases appeared to have been made in a satisfactory manner.

During a tour on April 28, 1981, the NRC inspector observed that the oil reservoir on snubber HRS-244 on the Hot Reheat System was empty. This was reported to the licensee who declared the snubber inoperable at 0930 as required by Technical Specification 4.3.10. The Technical Specification allows continued operation for a period of 72 hours. Repairs were made to the snubber and it was declared operable at 1520 on April 29, 1981. During this same tour the NRC inspector also observed that one of the mechanical snubbers (VSR-124) which is associated with the electromatic relief valve (V-22168 a non-safety related valve) on loop 2 was broken. This was also reported to the licensee who isolated the valve from the remainder of the loop 2 system. Replacement parts for the International Nuclear Safeguards Corporation Snubber, size MSVA-3, were being sought at the end of the report period.

No violations or deviations were identified.

#### 4. Surveillance (Monthly)

The NRC inspector reviewed all aspects of surveillance testing involving safety-related systems. The review included observation and review relative to Technical Specification requirements. The surveillance tests reviewed and observed were:

SR 5.8.2bc-M Radioactive Liquid Effluent System Instrument Function Test (Release #449)

SR 5.8.1abc-M Radioactive Gaseous Effluent System Test (Release #528)

SR 5.4.1.1.4.C-D/SR 5.4.1.4.2.C-D Linear Power Channel Heat Calibration

SR 5.6.1.b-SA Loss of Outside Power and Turbine Trip

SR 5.2.1c1-M/A PCRV Pressure Switches and Alarms Functional Test

No violations or deviations were identified.

#### 6. Review of Plant Operations

The NRC inspector reviewed aspects of facility operations to determine if they were being accomplished in accordance with regulatory requirements.

##### a. Training

It was determined by attending training lectures on the Hydraulic Power System, the Nuclear Instrumentation System, and the Plant Protective System that the training program objectives were met. By direct questioning of licensee personnel and training record review, it was determined that regulatory requirements, licensee commitments, and FSAR requirements for the training of nuclear power plant personnel were met.

##### b. Review and Audit

A Plant Operating Committee meeting was attended to determine compliance with the Technical Specification and other regulatory requirements.

No violations or deviations were identified.

7. Report Review

The NRC inspector reviewed the Monthly Operating Information Report for March 1981.

No violations or deviations were identified.

8. Review of Licensee Event Reports

The NRC inspector reviewed licensee event reporting activities to verify that they were in accordance with Technical Specification, Section 7, including identification details, corrective action, review and evaluation of aspects relative to operations and accuracy of reporting.

The following reports were reviewed by the inspector:

79-12	80-64	81-04
79-13	80-66	81-12
79-44	80-71	81-13
80-06	80-75	81-15
80-45	80-76	81-18
80-59	80-77	81-19
80-63	80-02	81-20

9. IE Circulars

The NRC inspector verified by record review, observation and discussion with the licensee, the action taken in response to IE Circulars.

The following circulars were reviewed:

- 80-04      Securing of Threaded Locking Devices on Safety-Related Equipment. The licensee has reviewed their maintenance procedures and found that securing of locking devices is addressed in these procedures.
- 80-12      Valve Shaft to Actuator Key May Fall Out of Place When Mounted Below Horizontal Axis. The licensee does not have Bettis-Pratt type connectors at Fort St. Vrain. However, an inspection was made of similar connections and no indications of problems were found.

80-18 10 CFR 50.59 Safety Evaluation for Changes to Radioactive Waste Treatment Systems. The licensee has reviewed their procedures for changes to plant systems and found that the requirements for safety evaluations as outlined in the circular are contained in their procedures.

No violations or deviations were identified.

10. TI 2515/42 Procedures and Staffing

1.A.2.1 Modify Training

The NRC inspector has verified that training in accordance with the requirements of NUREG-0578 for training to mitigate core damage was initiated on March 2, 1981, and completed April 6, 1981. A review of the training records verified that Operations, Technical Advisors, Health Physicists, Results and Management were included in the training.

No violations or deviations were identified.

11. Exit Interview

Exit interviews were conducted at the end of various segments of this inspection with Mr. D. Warembourg (Manager, Nuclear Production) and/or other members of the Public Service Company staff. At the interviews, the inspector discussed the findings indicated in the previous paragraphs. The licensee acknowledged these findings.