



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 68 TO FACILITY OPERATING LICENSE NO. DPR-32
AND AMENDMENT NO. 68 TO FACILITY OPERATING LICENSE NO. DPR-37
VIRGINIA ELECTRIC AND POWER COMPANY
SURRY POWER STATION, UNIT NOS. 1 AND 2
DOCKET NOS. 50-230 AND 50-281

Introduction

By letter dated September 22, 1978, as supplemented January 9 and September 24, 1979, Virginia Electric and Power Company (the licensee) requested amendments to License Nos. DPR-32 and DPR-37 for the Surry Power Station, Unit Nos. 1 and 2. These proposed amendments relate to Containment Leakage Testing, Appendix J to 10 CFR Part 50. This request also asked for an exemption to certain provisions of Appendix J related to personnel air lock testing.

Discussion

On August 4, 1975[1], the NRC requested the licensee to review its containment leakage testing program for Surry Power Station, Units 1 and 2, and the associated technical specifications, for compliance with the requirements of Appendix J to 10 CFR Part 50.

Appendix J to 10 CFR Part 50 was published on February 14, 1973. Since by this date there were already many operating nuclear plants and a number more in advanced stages of design or construction, the NRC decided to have these plants re-evaluated against the requirements of this new regulation. Therefore, beginning in August 1975, requests for review of the extent of compliance with the requirements of Appendix J were made of each licensee. Following the initial responses to these requests, NRC staff positions were developed which would assure that the objectives of the testing requirements of the above cited regulation were satisfied. These staff positions have since been applied in our review of the submittals filed by the licensee for the Surry Power Station, Units 1 and 2. The results of our evaluation are provided below.

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Evaluation

Our consultant, the Franklin Research Center (FRC), has reviewed the licensee's submittals [2, 3, 4, 5] and prepared the attached evaluation of containment tests for Surry Power Station, Units 1 and 2. We have reviewed this evaluation and concur in its bases and findings.

Based on our review of the attached technical evaluation report as prepared by the FRC, the following conclusions are made regarding the Appendix J review for Surry Power Station, Units 1 and 2:

1. The licensee's request for exemption from the requirement of Appendix J regarding the containment air lock testing is found to be no longer necessary because of the revision to Section III.D.2 of Appendix J (effective October 22, 1980). However, the licensee's proposed approach as stated in Reference 4 should be revised to include the following requirements:

Within 72 hours after use of the airlock, the seals will be tested at the peak calculated accident pressure to verify that they are properly seated.

We have discussed this change with the licensee and the licensee agrees and these words have been added to the Technical Specifications (T.S.)

2. The licensee's proposed changes to T.S. 4.4.A through 4.4.D (Reference 3) are found to be acceptable. However, due to the Appendix J revision, the exception made for the personnel hatch is no longer necessary, and the proposed T.S. 4.4.A and T.S. 4.4.D should be worded as follows.

Proposed T.S. 4.4.A

"Periodic and post-operational integrated leakage rate tests of the containment shall be performed in accordance with the requirements of 10 CFR Part 50, Appendix J, 'Reactor Containment Leakage Testing For Water Cooled Power Reactors.'"

Proposed T.S. 4.4.D

"The reset schedules for Type A, B and C tests will be in accordance with Section III.D of Appendix J."

In addition, specific references to the Federal Register Notices have been deleted.

Environmental Consideration

We have determined that the amendments do not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendments involve an action which is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR §51.5(d)(4), that an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of these amendments.

Conclusion

We have concluded, based on the considerations discussed above, that: (1) because the amendments do not involve a significant increase in the probability or consequences of accidents previously considered and do not involve a significant decrease in a safety margin, the amendments do not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of these amendments will not be inimical to the common defense and security or to the health and safety of the public.

Date: May 19, 1981

REFERENCES

- [1] NRC Generic Letter from Mr. Karl Goller, Acting Director for Operating Reactors, to Virginia Electric and Power Company, dated August 4, 1975.
- [2] VEPCO letter from C. M. Stallings to R. W. Reid, Chief, ORB-4, dated October 20, 1975.
- [3] VEPCO letter from C. M. Stallings to H. R. Denton, Director, Office of Nuclear Reactor Regulation, dated September 22, 1978, forwarding Proposed Technical Specification Change No. 69.
- [4] VEPCO letter from C. M. Stallings to H. R. Denton, Director, Office of Nuclear Reactor Regulation, dated January 9, 1979.
- [5] VEPCO letter from C. M. Stallings to H. R. Denton, Director, Office of Nuclear Reactor Regulation, dated September 24, 1979.

TECHNICAL EVALUATION REPORT

CONTAINMENT LEAKAGE RATE TESTING

VIRGINIA ELECTRIC AND POWER COMPANY
SURRY UNITS 1 AND 2

NRC DOCKET NO. 50-280, 50-281

NRC TAC NO. 08638, 08639

NRC CONTRACT NO. NRC-03-79-118

FRC PROJECT C5257

FRC TASK 50, 51

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