

UNITED STATES NUCLEAR REGULATORY COMMISSION WASHINGTON, D. C. 20555

GEORGIA POWER COMPANY OGLETHORPE POWER CORPORATION MUNICIPAL ELECTRIC AUTHORITY OF GEORGIA CITY OF DALTON, GEORGIA

DOCKET NO. 50-321

EDWIN I. HATCH NUCLEAR PLANT, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 35 License No. DPR-57

- 1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Georgia Power Company, et al., (the licensee) dated May 4, 1981, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
- Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C.(2) of Facility Operating License No. DPR-57 is hereby amended to read as follows:
 - (2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 85, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications. 81060300001 FOR THE NUCLEAR REGULATORY COMMISSION

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John F. Stolz, Chief Operating Reactors Branch #4 Division of Licensing

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Attachment: Changes to the Technical Specifications

Date of Issuance: May 26, 1981

ATTACHMENT TO LICENSE AMENDMENT NO. 35

FACILITY OPERATING LICENSE NO. DPR-57

DOCKET NO. 50-321

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by Amendment number and contain vertical lines indicating the area of change.

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Remove	Insert
3.6-9a	3.6-9a
3.7-26	3.7-26
3.7-27	3.7-27

LIMITING CONDITIONS FOR OPERATION

SURVEILLANCE REQUIREMENTS

- 4.6.H. Relief/Safety Valves (Continued)
 - Operability of Tail Pipe Pressure Switches
 - a. Functional Test:
 - At each scheduled outage greater than 72 hours during which entry is made into the primary containment, if not performed within the previous-31 days.
 - b. Calibration and verifying the setpoint to be 30 ± 5 psig at least once per 18 months.

I. Jet Pumps

Whenever both recirculating pumps are operating with the reactor in the Start & Hot Standby or Run Mode, jet pump operability shall be checked daily by verifying that the following conditions do not occur simultaneously.

 The two recirculation loops have a flow imbalance of 15% or more when the pumps are operated at the same speed.

.I. Jet Pumps

Whenever the reactor is in the Start & Hot Standby or Run Mode with both recirculating pumps operating, all jet pumps shall be operable. If it is determined that a jet pump is inoperable, an orderly shutdown shall be initiated and the reactor shall be in the Cold Shutdown Condition with 24 hours.

Table 3.7-4 (Concluded)

Primary Containment Testable Isolation Valves

Penetration Number	Valve Designation	Notes
X-220	P33-F006 & F014	(1) (2) (4) (5) (9)
X-220	T48-F318 & F326	
X-220	T48-F332A & F333A	•
X-220	T48-F332B & F333B	•
X-220	T48-F338 & F339	•
X-218A	G51-F002	(1) (2) (4) (5) (10)
X-218A	G51-0001	(1) (2) (4) (5) (13)
X-218A	G51-FQ16	(1) (2) (4) (5) (14)
X-218A	G51-D004	(1) (2) (4) (5) (13)

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Amendment No. \$3, 85

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Notes For Tables 3.7-2 through 3.7-4

- Test duration for all valves and penetrations listed will generally exceed one hour.
- (2) Test pressures are at least 59 psig for all valves and penetrations except MSIV's which are tested at 28 psig.
- (3) MSIV acceptable leakage limit is 11.5 scfh/valve of air.
- (4) The total acceptable leakage for all valves and penetrations other than the SIV's is 0.6 La.
- (5) Local leak tests on all testable isolation valves shall be performed each major refueling shutdown but in no case at intervals greater than 2 years.
- (6) Local leak tests on all testable penetrations shall be performed each major refueling shutdown but in no case at intervals greater than 2 years.
- (7) The personnel air lock shall be tested at intervals not to exceed 6 months.
- (8) The personnel air lock door seals are tested at 10 psig after each opening.
- (9) Identifies isolation valves that are tested by applying pressure between the inboard and outboard isolation valves. Inboard valve is not tested in the direction required for isolation but will have equivalent or more conservative leakage results.
- (10) identifies isolation values that are tested by applying pressure between the isolation value and a manually operated value such that the isolation value is tested in the direction required for isolation.
- (11) Identifies isolation values that are tested by applying pressure between the isolation values and other system values. Isolation value, not tested in the direction required for isolation will have equivalent or more conservative results.
- (12) The RHR system remains water filled post-LOCA. Isolation valves are tested with water.
- (13) Identifies blind flange that is tested by applying pressure between the flange and a manually operated valve such that the flange is tested in the direction required for isolation.
- (14) Identifies an isolation valve that is tested by applying pressure between the manually operated isolation valve and a blind flange. The isolation valve is not tested in the direction required for isolation but will have equivalent or more conservative results.