DSB

DAIRYLAND POWER COOPERATIVE · P.O. BOX 817 · 2615 EAST AV. SOUTH · LA CROSSE, WISCONSIN 54601
(608) 788-4000

May 19, 1981

In reply, please refer to LAC-7555

DOCKET NO. 50-409

Mr. James G. Keppler, Director
U. S. Nuclear Regulatory Commission
Directorate of Regulatory Operations
Region III
799 Roosevelt Road
Glen Ellyn, Illinois 60137

SUBJECT: DAIRYLAND POWER COOPERATIVE
LA CROSSE BOILING WATER REACTOR (LACBWR)
PROVISIONAL OPERATING LICENSE NO. DPR-45
REPORTABLE OCCURRENCE NO. 81-06

Reference: (1) LACBWR Technical Specifications, Section 6.9.1.9.c
(2) LACBWR Technical Specifications, Section 6.9.1.8.c

Dear Mr. Keppler:

In accordance with References 1 and 2, this is to inform you of an inadequacy in the implementation of administrative and procedural controls which caused a reduction in the degree of redundancy provided in reactor protection systems and potential degradation of primary containment.

On May 17, 1981, a LACBWR Supervisor discovered a modification had been made to the Containment Building pressure sensing line leading to Pressure Switch 37-35-702 which activates 1B High Pressure Core Spray Pump (HPCS) and 1B High Pressure Service Water (HPSW) Alternate Core Spray (ACS) Pump and sends half the opening signal to the AC Alternate Core Spray Valve on high Containment Building pressure of 5 psig. The installation had been made without an approved Maintenance Request or Facility Change. The modification consisted of an additional pressure switch, two additional valves and 3/8 in. copper tubing. The modification had been assembled and leak tested at 60 psig without leakage prior to being installed.

During the installation on April 1, 1981, which took less than one hour, the valve between the containment wall and the Pressure Switch 37-35-702 was closed for approximately one minute. If the Containment Building had become pressurized during this time, which it did not, the 1B HPCS Pump, 1B HPSW/ACS Pump and AC ACS Valve would not

have received a high pressure actuating signal. The 1A HPCS Pump, 1A HPSW/ACS Pump and DC ACS Valve and other starting signals, were unaffected by the installation. Therefore, the installation process reduced the degree of redundancy in these systems, but would not have prevented system actuation if required.

A leakage test was not performed after installation. Therefore, it cannot be proven that the swagelok fitting connecting the modification and the sensing line would not leak in excess of Technical Specification limits when internally exposed to 52 psig and that Containment Integrity exists. The added valves were found in the closed position and have since been tagged to ensure they are not opened. A leakage test will be performed on the modification in the near future.

Prompt notification of this event was made to the NRC Operations Center at 2315 on May 18, 1981, to the Resident Inspectors at approximately 0930 on May 19, 1981, and to Mr. Len McGregor of Region III at 1000 on May 19.

If there are any questions concerning this report, please contact us.

Very truly yours,

DAIRYLAND POWER COOPERATIVE

Frank Linder

Frank Linder, General Manager

FL:LSG:af

cc: Director, Office of Management Information (3)
and Program Control
U. S. Nuclear Regulatory Commission
Washington, D. C. 20555

NRC Resident Inspectors

NRC FORM 366
(7-77)

U. S. NUCLEAR REGULATORY COMMISSION

LICENSEE EVENT REPORT

CONTROL BLOCK: (1) (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

0 1 | (2) LICENSEE CODE | (3) LICENSE NUMBER | (4) LICENSE TYPE | (5) CAT

CON'T

0 1 | REPORT SOURCE (6) | 0 5 0 0 0 4 0 9 (7) | 0 5 / 1 7 5 / (8) | 0 5 / 1 9 5 / (9)
DOCKET NUMBER | EVENT DATE | REPORT DATE

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

0 2

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0 9 | SYSTEM CODE (11) | CAUSE CODE (12) | CAUSE SUBCODE (13) | COMPONENT CODE (14) | COMP. SUBCODE (15) | VALVE SUBCODE (16)

(17) LER/RO REPORT NUMBER | EVENT YEAR 8 1 | SEQUENTIAL REPORT NO. 0 0 6 | OCCURRENCE CODE | REPORT TYPE | REVISION NO.

ACTION TAKEN (18) | FUTURE ACTION (19) | EFFECT ON PLANT (20) | SHUTDOWN METHOD (21) | HOURS (22) | ATTACHMENT SUBMITTED (23) | NPRD-4 FORM SUB. (24) | PRIME COMP. SUPPLIER (25) | COMPONENT MANUFACTURER (26)

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

1 0

1 1

1 2

1 3

1 4

1 5 | FACILITY STATUS (28) | % POWER (29) | OTHER STATUS (30) | METHOD OF DISCOVERY (31) | DISCOVERY DESCRIPTION (32)

1 6 | ACTIVITY RELEASED (33) | CONTENT OF RELEASE (34) | AMOUNT OF ACTIVITY (35) | LOCATION OF RELEASE (36)

PERSONNEL EXPOSURES

1 7 | NUMBER (37) | TYPE (38) | DESCRIPTION (39)

PERSONNEL INJURIES

1 8 | NUMBER (40) | DESCRIPTION (41)

LOSS OF OR DAMAGE TO FACILITY (43)

1 9 | TYPE (42) | DESCRIPTION

PUBLICITY

2 0 | ISSUED (44) | DESCRIPTION (45)

NRC USE ONLY

NAME OF PREPARER _____ PHONE: _____

GPO 911-926