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SACRAMENTO MUNICIPAL UTILITY DISTRICT □ 6201 S Street, Box 15830, Sacramento, California 95813; (916) 452-3211

May 22, 1981

DIRECTOR OF NUCLEAR REACTOR REGULATION
ATTENTION DARRELL G EISENHUT DIRECTOR
DIVISION OF LICENSING
U S NUCLEAR REGULATORY COMMISSION
WASHINGTON DC 20555

DOCKET 50-312
RANCHO SECO NUCLEAR GENERATING
STATION UNIT NO 1
THERMAL SHOCK TO REACTOR PRESSURE VESSEL

The Sacramento Municipal Utility District has received your letter of April 20, 1981 requesting information on the effects of thermal shock to reactor pressure vessels. As Chairman of the Babcock & Wilcox Owners Regulatory Response Group, I responded on May 12, 1981 and provided you with a letter report on reactor vessel brittle fracture concerns in Babcock & Wilcox operating plants and a summary of the Electric Power Research Institute programs pertaining to brittle fracture. This response provided the information requested in your letter and during meetings with NRC staff on March 31, 1981 and April 29, 1981.

We have determined that the information provided in my letter of May 12, 1981 is applicable to Rancho Seco Unit No. 1 and support the conclusions presented therein. One of the conclusions was that the analysis presented in BAW-1648, "Thermal Mechanical Report - Effect of HPI on Vessel Integrity for Small Break LOCA Event with Extended Loss of Feedwater", represents a bounding cooldown event. We commit to perform further plant specific analyses to demonstrate that considerable time exists before there are any concerns over brittle fracture during such an event at Rancho Seco Unit No. 1. These analyses will consider the actual borated water storage tank temperature of 80° and limiting weld locations. Also, we have previously committed to install an upgraded auxiliary feedwater control system which will provide safety grade control of auxiliary feedwater, including

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a means to prevent primary system overcooling. These controls will be installed during the first extended outage following equipment delivery in 1982. This system was described in our letter of November 17, 1980. We further commit to participate in generic Owners Group activities which develop in the area of reactor vessel thermal shock. We feel confident that these actions will provide adequate justification for operation of Rancho Seco Unit No. 1 for its design lifetime.

John J. Mattimoe

John J. Mattimoe
Asisstant General Manager
and Chief Engineer