

FACILITY OPERATING LICENSE NO. DPR-73

DOCKET NO. 50-320

Replace the following pages of the proposed Appendix "A" Technical Specifications with the enclosed pages as indicated. The revised pages contain vertical lines indicating the area of change. The corresponding overleaf pages are also provided to maintain document completeness.

Pages

2-1
B2-1
3.3-7
3.4-1
6-15
6-16

2.0 SAFETY LIMITS

2.1 SAFETY LIMITS

REACTOR COOLANT SYSTEM PRESSURE

2.1.3 Not applicable.

2.1 SAFETY LIMITS

BASES

2.1.3 REACTOR COOLANT SYSTEM PRESSURE

As a result of the March 28, 1979 accident, the pressure retaining ability of the reactor coolant system is uncertain. Therefore, a meaningful reactor coolant system pressure Safety Limit can not be established. However, the reactor coolant system pressure has been reduced to approximately 90 psig. Furthermore, since Technical Specification 3.4.9.1 establishes 600 psig as a Limiting Condition for Operation for the reactor coolant system, no further limitations are required.

LIMITING CONDITIONS FOR OPERATION

REMOTE SHUTDOWN INSTRUMENTATION

3.3.3.5 The remote shutdown monitoring instrumentation channels shown in Table 3.3-9 shall be OPERABLE with readouts displayed external to the control room.

APPLICABILITY: RECOVERY MODE.

ACTION:

With the number of OPERABLE remote shutdown monitoring channels less than required by Table 3.3-9, notify the NRC within 24 hours and restore the inoperable channel to OPERABLE status within 30 days.

POST-ACCIDENT INSTRUMENTATION

3.3.3.6 The post-accident monitoring instrumentation channels shown in Table 3.3-10 shall be OPERABLE.

APPLICABILITY: RECOVERY MODE.

ACTION:

With the number of OPERABLE post-accident monitoring channels less than required by Table 3.3-10, restore the inoperable channel(s) to OPERABLE status within 8 hours.

CHLORINE DETECTION SYSTEMS

3.3.3.7 Two chlorine detection systems, with their alarm/trip setpoints adjusted to actuate at a chlorine concentration of less than or equal to 5 ppm, shall be OPERABLE:

- a. One at the air intake tunnel, and
- b. One at the control room air supply duct.

APPLICABILITY: RECOVERY MODE.

ACTION:

With one or more chlorine detection systems inoperable, within 1 hour initiate and maintain operation of the control room emergency ventilation system in the recirculation mode of operation; restore the inoperable detection system to OPERABLE status within 30 days.

ADMINISTRATIVE CONTROLS

SPECIAL REPORTS

6.9.2 Special reports shall be submitted to the Director of the Office of Inspection and Enforcement Regional Office within the time period specified for each report.

6.10 RECORD RETENTION

6.10.1 The following records shall be retained for at least five years:

- a. Records of sealed source and fission detector leak tests and results.
- b. Records of annual physical inventory of all sealed source material of record.
- c. Records of changes made to the procedures required by Specifications 6.8.1 d. and e.

6.10.2 The following records shall be retained as long as the Licensee has an NRC license to operate or possess the Three Mile Island facility.

- a. Records and logs of unit operation covering time interval at each power level.
- b. Records and logs of principal maintenance activities, inspections, repair and replacement of principal items of equipment related to nuclear safety and radioactive waste systems.
- c. ALL REPORTABLE OCCURRENCES submitted to the Commission.
- d. Records of surveillance activities, inspections and calibrations required by these Technical Specifications.
- e. Records of changes made to the procedures required by Specifications 6.8.1 a., b., c., f. and g.
- f. Reports required by 6.9.1.6 and 6.9.1.10.
- g. Records of radioactive shipments.
- h. Records and logs of radioactive waste systems operations.
- i. Records and drawing changes reflecting facility design modifications made to systems and equipment described in the Safety Analysis Report.
- j. Records of new and irradiated fuel inventory, fuel transfers and assembly burnup histories.

LIMITING CONDITIONS FOR OPERATION

3.4 REACTOR COOLANT SYSTEM

REACTOR COOLANT LOOPS

3.4.1 The Reactor Coolant System shall be operated in accordance with procedures approved pursuant to Specification 6.8.2.

APPLICABILITY: RECOVERY MODE.

ACTION:

None except as provided in Specification 3.0.3.

SAFETY VALVES

3.4.3 All pressurizer code safety valves shall be OPERABLE with a lift setting of 2435 PSIG \pm 1%.*

APPLICABILITY: RECOVERY MODE.

ACTION:

None except as provided in Specification 3.0.3.

3.4.9 PRESSURE/TEMPERATURE LIMITS

REACTOR COOLANT SYSTEM

3.4.9.1 The Reactor Coolant System shall be maintained at a T_{avg} of less than 280°F and at a pressure of less than 600 psig.

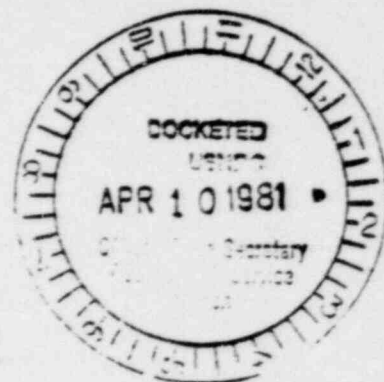
APPLICABILITY: When fuel is in the reactor pressure vessel.

ACTION:

With the Reactor Coolant System pressure exceeding 600 psig, immediately reduce the Reactor Coolant System pressure to within its limit.

*The lift setting pressure shall correspond to ambient conditions of the valve at nominal operating temperature and pressure.

UNITED STATES OF AMERICA
NUCLEAR REGULATORY COMMISSION



BEFORE THE ATOMIC SAFETY AND LICENSING BOARD

In the Matter of
METROPOLITAN EDISON COMPANY, et al.
(Three Mile Island Nuclear Station,
et al.)

Docket No. 50-320 OLA

April 9, 1981

SERVED APR 10 1981

ORDER GRANTING JOINT MOTION
TO APPROVE STIPULATION



On April 3, 1981, this Atomic Safety and Licensing Board was served with a Joint Motion filed by several parties to the captioned proceeding requesting approval of the "Stipulation Regarding Settlement of Issues" attached thereto. According to the Joint Motion, the subject stipulation was entered into by Intervenor Steven Sholly and William Lochstat, the NRC Staff and the Licensee for the purpose of resolving all of the issues advanced by Mr. Sholly and one of the issues advanced by Mr. Lochstat relative to this proceeding. The stipulation contains the basis upon which an agreement was reached among the parties by which the issues in question are deemed settled.

The Licensing Board regards the Joint Motion and subject Stipulation as furthering the principles of settlement and compromise of NRC litigation. Accordingly, it is hereby ORDERED that:

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1. The Joint Motion is granted;
2. The "Request for Hearing" of Steven C. Sholly dated March 21, 1980, and supplement thereto, dated June 19, 1980, are deemed withdrawn and Mr. Sholly thereby dismissed from the proceeding;
3. Proposed Contention 1 contained in the supplement to the request for hearing of Mr. William Lochstet, dated June 17, 1980, is deemed withdrawn; and
4. Proposed Technical Specifications 2.1.3, 3.3.3.5, 3.4.9.1, 6.10.1, and 6.10.2 set forth in an attachment to the February 11, 1980 Order issued by the Director, Nuclear Reactor Regulation, shall be modified in accordance with the revisions specified in the proposed Amendment of Order attached to the "Stipulation Regarding Settlement of Issues" upon entry of that Amendment of Order.

FOR THE ATOMIC SAFETY AND LICENSING BOARD

John F. Wolf
Administrative Judge
John F. Wolf, Esq., Chairman

Dated at Bethesda, Maryland
this 9th day of April, 1981.