

ENCLOSURE 1

Arkansas Power and Light Company
Arkansas Nuclear One, Unit No. 2
Docket No. 50-368
License No. NPF-6

NOTICE OF VIOLATION

Based on the results of an NRC inspection conducted during the period of January 22 - February 21, 1981, and in accordance with the Interim Enforcement Policy, 45 FR 66754 (October 7, 1980), the following violations were identified:

1. Technical Specification 2.2.1 "Reactor Trip Setpoints" states, in part: "With a reactor protective instrumentation setpoint less conservative than the value shown in the Allowable Values column of Table 2.2-1, declare the channel inoperable and apply the applicable ACTION statement requirement of Specification 3.3.1.1 until the channel is restored to OPERABLE status with its trip adjusted consistent with the Trip Setpoint value."

Technical Specification 3.3.1.1 states, in part: "Each reactor protective instrumentation channel shall be demonstrated OPERABLE by the performance of the CHANNEL CHECK, CHANNEL CALIBRATION and CHANNEL FUNCTIONAL TEST operations for the MODES and at the frequencies shown in Table 4.3-1."

Contrary to the above, during the period of January 1980 through January 1981, the licensee did not demonstrate that each reactor protective instrument channel was OPERABLE for the MODES and at the frequencies shown in Table 4.3-1 of Technical Specification 3.3.1.1. Specifically, during the period of January 1980 through January 1981, the licensee did not compare the reactor trip setpoints with the Allowable Values of Table 2.2-1 of Technical Specification 2.2.1 when performing the CHANNEL CALIBRATION and CHANNEL FUNCTIONAL TEST operations on the reactor protective instrumentation channels.

This constitutes a Severity Level V Violation. (Supplement I.E.)(368/8103-01)

2. 10 CFR 20.201(b) "Surveys" states, "Each licensee shall make or cause to be made such surveys as may be necessary for him to comply with the regulations in this part."

Contrary to the above, the licensee failed to evaluate the radiation levels associated with the high radiation area at the 317' level of the Unit 2 auxiliary building. Specifically, an entry was made into this area (317' level) by an NRC inspector and a health physics technician on January 29, 1981. The escort allowed access to the high radiation area without adequate knowledge of the radiological hazards in the area and without the possession of a radiation survey instrument to determine the radiological hazards.

This constitutes a Severity Level IV Violation. (Supplement IV.D.1)
(368/8103-02)

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3. Criterion V to Appendix B of 10 CFR states: "Activities affecting quality shall be prescribed by documented instructions, procedures, or drawings, of a type appropriate to the circumstances and shall be accomplished in accordance with these instructions, procedures or drawings."

Small pipe isometric drawing 2HBC-92-2 shows the locations of six seismic Class I critical pipe hangers for line number 2HBC-92-2".

Contrary to the above, on January 30, 1981, one of the required pipe supports was not installed on line 2HBC-92-2". This line is in the loop I service water return line from 2VUC-2A, a switchgear room cooler.

This is a Severity Level IV Violation. (Supplement I.D.3) (368/8103-03)

Pursuant to the provisions of 10 CFR 2.201, Arkansas Power and Light Company is hereby required to submit to this office within 25 days of the date of this Notice, a written statement or explanation in reply, including (1) the corrective steps which have been taken and the results achieved; (2) corrective steps which will be taken to avoid further violations; and (3) the date when full compliance will be achieved. Under the authority of Section 182 of the Atomic Energy Act of 1954, as amended, this response shall be submitted under oath or affirmation.

Dated 3/16/81

G. L. Madsen
G. L. Madsen, Chief
Reactor Projects Branch