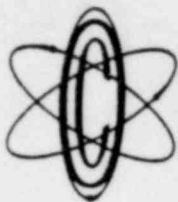


OYSTER CREEK



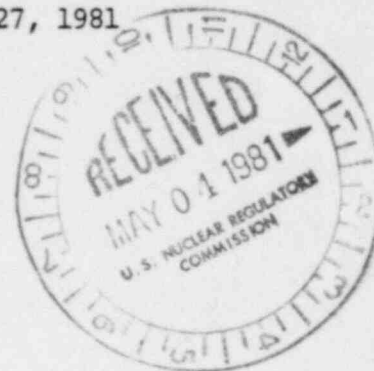
NUCLEAR GENERATING STATION

JCP&L GPU

Jersey Central Power & Light
Company is a Member of the
General Public Utilities System

(609) 693-6000 P.O. BOX 388 • FORKED RIVER • NEW JERSEY • 08731

April 27, 1981



Mr. Boyce H. Grier, Director
Office of Inspection and Enforcement
Region I
United States Nuclear Regulatory Commission
631 Park Avenue
King of Prussia, Pennsylvania 19406

Dear Mr. Grier:

SUBJECT: Oyster Creek Nuclear Generating Station
Docket No. 50-219
Licensee Event Report
Reportable Occurrence No. 50-219/81-19/3L

This letter forwards three copies of a Licensee Event Report to report Reportable Occurrence No. 50-219/81-19/3L in compliance with paragraph 6.9.2.b.3 of the Technical Specifications.

Very truly yours,

Ivan R. Finfrock, Jr.
Ivan R. Finfrock, Jr.
Vice President - JCP&L
Director - Oyster Creek

IRF:dh
Enclosures

cc: Director (40 copies)
Office of Inspection and Enforcement
United States Nuclear Regulatory Commission
Washington, D.C. 20555

Director (3)
Office of Management Information
and Program Control
United States Nuclear Regulatory Commission
Washington, D. C. 20555

NRC Resident Inspector (1)
Oyster Creek Nuclear Generating Station
Forked River, N. J.

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OYSTER CREEK NUCLEAR GENERATING STATION
Forked River, New Jersey 08731

Licensee Event Report
Reportable Occurrence No. 50-219/81-19/3L

Report Date

April 27, 1981

Occurrence Date

March 28, 1981

Identification of Occurrence

Failure to perform a surveillance test as required by Technical Specifications, paragraph 4.1 Table 4.1.1.

This event is considered to be a reportable occurrence as defined in the Technical Specifications, paragraph 6.9.2.b.3.

Conditions Prior to Occurrence

Routine Shutdown

Description of Occurrence

On March 28, the plant was progressing towards cold shutdown to correct an increasing leak rate in the Primary Containment. During the normal shutdown sequence, the IRM Calibration was not performed as required. When the IRM's were inserted sufficiently for the calibration, power level was too low for the calibration to be performed. The correlation used to calculate reactor power is inaccurate below 10% power.

Apparent Cause of Occurrence

Procedure - Procedure 203.2 did not adequately specify the power level at which to perform the calibration.

Analysis of Occurrence

The purpose of the IRM System is to provide a means of monitoring reactor power level between the APRM and SRM ranges, and to provide SCRAM protection in the event of a reactivity excursion.

The system was not rendered inoperable, but was possibly out of calibration. During a shutdown, the contribution to the heat balance by decay heat makes the calibration less accurate than during a startup. Due to this consideration and the continuing process of shutdown at the time of occurrence, the safety significance of this event is considered to be minimal.

Corrective Action

The calibration was performed during the startup on March 31, 1981.

Both the IRM Calibration procedure and the Plant Cooldown from Hot Standby to Cold Shutdown procedure will be revised prior to startup to specify the power level at which to perform the IRM Calibration.

Failure Data

Not Applicable.