

CONNECTICUT YANKEE ATOMIC POWER COMPANY

HADDAM NECK PLANT

HADDAM, CONNECTICUT

MONTHLY OPERATING REPORT NO. 81-3

FOR THE MONTH OF

MARCH 1981

8104280536

## PLANT OPERATIONS

The following is a chronological description of plant operations for the month of March 1981:

- 3/1/81 The plant was operating at 100 percent full power (604 MWe) at the beginning of this report period and remained at full power until load was reduced for condenser leak check on March 9, 1981.
- 3/3/81 Backwashed C & D condenser waterboxes
- 3/4/81 Made air charge to containment 1315 to 1420
- 3/6/81 Passed 54 billion KW mark on gross generation at 1145 hours. Backwashed all condenser waterboxes
- 3/7/81 #3 RCP seal water return flow indicator failed low at 0358 hours. I & C replaced power supply and returned #3 RCP seal water return indication to service at 0653 hours.
- 3/9/81 Commenced load reduction for condenser leak check at 0958 hours. Load at 400 MWe and shut down "B" circulating pump at 1123 hours.
- Received report of fire in Primary Auxiliary Building at 1132 hours. Fire in roof above boric acid storage room extinguished with water
- Restarted "B" circulating pump - no leaks in B waterbox at 1325 hours. Shut down "C" circulating pump at 1340 hours. Plugged one tube in C waterbox. Restarted "C" circulating pump at 1546 hours. Shut down "D" circulating pump at 1602 hours - no leaks. Restarted "D" circulating pump at 1755 hours. Backwashed "A" condenser at 1810 hours. Reduced load to 350 MWe for turbine stop and control valve test at 1839 hours. Completed valve test and commenced load increase at 1930 hours. Plant was at full load at 2310 hours and remained at full power for the remainder of this report period.
- 3/19/81 Divers cleaned screenhouse trash racks 0850 hours to 1415 hours.
- 3/23/81 I & C vented reference leg on channel 3 pressurizer level. Level indication from 53.8 to 51 percent.
- 3/25/81 Backwashed A & B condensers

SYSTEM OR COMPONENT	MALFUNCTION		EFFECT ON SAFE OPERATION	CORRECTIVE ACTION TAKEN TO PREVENT REPETITION	SPECIAL PRECAUTIONS TAKEN TO PROVIDE FOR REACTOR SAFETY DURING REPAIR
	CAUSE	RESULT			
Pressurizer Pressure Channel Pic - 401-1	Loose connector on Pic 401-1 (controller for PORV) Reason for loose connector was possibly jarring of MCB by workmen.	Loss of channel control on PORV. Opened PORV 570 and MOV 569	None: Pressur- izer pressure decreased ~25-30 PSIG	Connector taped to control- ler. Checked all other connectors for tightness.	Use of portions of SUR 5.2-4. Redundant indication of pressurizer pressure were available.
"B" service water filter backwash line elbow	Erroded elbow Reason: age	Service water leak	None	Replaced elbow	None required
Cable vault Main CO <sub>2</sub> system. SUR 5.5-20	Main CO <sub>2</sub> system bank time delay did not function Reason: unknown  Fan dampners did not close. Reason: Loose shaft set screws.	Automatic initia- tion of CO <sub>2</sub> system inoperative.	None	Disassembled and cleared time delay. Replaced fan dampner shaft set screws. Retested system	None required
Switch gear Halon system SUR 5.5-23	Automatic initia- tion of Halon system failed. Reason: interfer- ence between horn circuit and sol- enoid initiating circuit.	Automatic initia- tion of Halon system inopera- tive.	None	Installed 3 Varistor in circuit. Retested system	None required

POOR ORIGINAL

CONNECTICUT YANKEE  
 REACTOR COOLANT DATA  
 MONTH: MARCH 1981

REACTOR COOLANT ANALYSIS	MINIMUM	AVERAGE	MAXIMUM
PH @ 25 DEGREES C	6.17E+00	6.36E+00	6.72E+00
CONDUCTIVITY (UMHOS/CM)	8.30E+00	1.34E+01	1.80E+01
CHLORIDES (PPM)	<4.00E-02	<4.00E-02	<4.00E-02
DISSOLVED OXYGEN (PPB)	<5.00E+00	<5.00E+00	<5.00E+00
BORON (PPM)	4.71E+02	5.18E+02	5.64E+02
LITHIUM (PPM)	9.00E-01	1.53E+00	2.00E+00
TOTAL GAMMA ACT. (UC/ML)	1.66E+00	3.66E+00	5.36E+00
IODINE-131 ACT. (UC/ML)	1.75E-02	2.04E-02	2.37E-02
I-131/I-133 RATIO	6.84E-01	7.82E-01	8.80E-01
CRUD (MG/LITER)	0.00E-01	0.00E-01	0.00E-01
TRITIUM (UC/ML)	3.07E+00	3.43E+00	3.82E+00
HYDROGEN (CC/KG)	2.04E+01	2.22E+01	2.63E+01

AERATED LIQUID WASTE PROCESSED(GALLONS): 8.97E+04  
 WASTE LIQUID PROCESSED THROUGH BORON RECOVERY(GALLONS): 0.00E-01  
 AVERAGE PRIMARY LEAK RATE(GALLONS PER MINUTE): 2.34E-01  
 PRIMARY TO SECONDARY LEAK RATE(GALLONS PER MINUTE): 0.00E+00

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AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-213

UNIT Conn. Yankee  
Haddam Neck

DATE April 12, 1981

COMPLETED BY Reactor Engineering

TELEPHONE (203) 267-2556

MONTH: March 1981

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	576
2	577
3	577
4	577
5	577
6	577
7	578
8	578
9	480
10	578
11	580
12	580
13	581
14	580
15	580
16	580

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
17	580
18	580
19	579
20	579
21	580
22	580
23	579
24	579
25	580
26	580
27	580
28	580
29	580
30	580
31	579

**POOR ORIGINAL**

INSTRUCTIONS

On this format, list the average daily unit power level in MWe-Net for each day in the reporting month. Complete the nearest whole megawatt.

UNIT SHUTDOWNS AND POWER REDUCTIONS

50-213

POCKET NO. 50-213  
 UNIT NAME Conn. Yankee  
 DATE April 12, 1981  
 COMPLETED BY Reactor Engineering  
 TELEPHONE (203) 267-3556

REPORT MONTH March 1981

No.	Date	Type	Duration (Hours)	Reason	Method of Shutting Down Reactor	License Event Report #	System Code	Component Codes	Cause & Corrective Action to Prevent Recurrence
81-6	810309	F	N/A	A	N/A	N/A	ZZ	ZZZZZZZ	Load reduction for valve test and tube plugging in Condenser B; "C" water box

POOR ORIGINAL

F Forced  
 S Scheduled

Reason:  
 A-Equipment Failure (Explain)  
 B-Maintenance or Test  
 C-Refueling  
 D-Regulatory Restriction  
 E-Operator Training & License Expiration  
 F-Administrative  
 G-Operational Error (Explain)  
 H-Other (Explain)

Method:  
 1-Manual  
 2-Manual Scram  
 3-Automatic Scram  
 4-Other (Explain)

Exhibit G - Instructions for Preparation of Data Entry Sheets for License  
 E.g. Report (LFR) File (NUREG-0161)

Exhibit I - Name Source

// JOB 0000011111  
// XED 00B FX

\*\*\*\*\*NRC OPERATING STATUS REPORT COMPLETED BY REACTOR ENGINEERING\*\*\*\*\*

1. UNIT NAME.....(OWN, TAYLOR ATOMIC POWER CO.
2. REPORTING PERIOD..... MARCH 1981
3. LICENSED THERMAL POWER(MWT).....1825
4. NAMEPLATE RATING(GROSS MWE).....600.3
5. DESIGN ELECTRICAL FATIGUE(INET MWE).....580
6. MAXIMUM DEFERRABLE CAPACITY(GROSS MWE).....582
7. MAXIMUM DEFERRABLE CAPACITY(INET MWE).....555
8. IF CHANGES OCCUR IN CAPACITY RATINGS(LIENS 3 THROUGH 7) SINCE LAST REPORT, GIVE REASONS... NONE
9. POWER LEVEL TO WHICH RESTRICTED. IF ANY(NET MWE)..... NONE
10. REASON FOR RESTRICTION. IF ANY.... N/A

DOCKET NO. 50-213  
DATE APRIL 17, 1981  
COMPLETED BY..... REACTOR ENGINEERING  
TELEPHONE (203) 267-2556

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THIS REPORTING PERIOD      YR. TO DATE      CUMULATIVE TO DATE

11. HOURS IN REPORTING PERIOD	744.0	2160.0	116136.0 *
12. NUMBER OF HOURS THE REACTOR WAS CRITICAL	744.0	2152.1	100124.7 *
13. REACTOR RESERVE SHUTDOWN HOURS	0.0	9.1	1192.5 *
14. HOURS GENERATOR ON LINE	744.0	2143.0	95664.0 *
15. UNIT RESERVE SHUTDOWN HOURS	0.0	0.0	373.7
16. GROSS THERMAL ENERGY GENERATED (MMH)	134920.	3811877.	165363270.
17. GROSS ELECTRICAL ENERGY GENERATED (MMH)	448344.	1265379.	54368577.
18. NET ELECTRICAL ENERGY GENERATED (MMH)	428528.	1207606.	51725214.
19. UNIT SERVICE FACTOR	100.0	99.2	82.4 *
20. UNIT AVAILABILITY FACTOR	100.0	99.2	82.7 *
21. UNIT CAPACITY FACTOR (USING MDC NET)	103.8	100.7	82.3 *
22. UNIT CAPACITY FACTOR (USING DER NET)	99.3	96.4	76.0 *
23. UNIT FORCED OUTAGE RATE	0.0	0.8	6.8 *

24. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE AND DURATION OF EACH) ... NONE

25. IF SHUTDOWN AT END OF REPORTING PERIOD, ESTIMATED DATE OF STARTUP... N/A

26. UNITS IN TEST STATUS (PRIOR TO COMMERCIAL OPERATION) ... NOT APPLICABLE

1. Name of facility

Connecticut Yankee Atomic Power Company

Scheduled date for next refueling shutdown.

September/October 1981

3. Scheduled date for restart following refueling

Approximately six to eight weeks.

4. (a) Will refueling or resumption of operation thereafter require a technical specification change or other license amendment?

No technical specification changes are anticipated at this time.

(b) If answer is yes, what, in general, will these be?

N/A

(c) If answer is no, has the reload fuel design and core configuration been reviewed by your Plant Safety Review Committee to determine whether any unreviewed safety questions are associated with the core reload (Ref. 10 CFR Section 50.59)?

When the above stated documents are received from the fuel vendor they will be reviewed in accordance with 10CFR50.59 to determine if any unreviewed safety questions are associated with the Core reload.

(d) If no such review has taken place, when is it scheduled?

N/A

5. Scheduled date(s) for submitting proposed licensing action and supporting information.

There are no scheduled dates because of (4) above.

6. Important licensing consideration associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures.

None

7. The number of fuel assemblies (a) in the core and (b) in the spent fuel storage pool.

(a) 157 (b) 389

8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned. in number of fuel assemblies.

1168

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9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity.

1994 to 1995