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2 8 9 14 15 25 26 30 37 CAT 36
LICENSER CODE LICENSE NUMBER LICENSE TYPE

REPORT SOURCE L 6 0 5 0 0 0 3 2 7 7 0 3 1 4 8 1 8 0 4 1 0 8 1 9

60 61 DOCKET NUMBER 68 69 EVENT DATE 74 75 REPORT DATE 80

07 | At 1204 CST with unit in mode 3 after a reactor trip, the steam-driven auxiliary feed-

03 | water pump 1A-S was declared inoperable when the operator experienced problems with the

04 | steam supply controls. The plant entered action statement "a" of LCO 3:7.1.2. There

05 | was no effect on public health or safety. There have been no previous occurrences.

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C H 9 10 11

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A 12 13

I N S T R U 13 14 18

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NO.
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COMPONENT
MANUFACTURER

10 The problems with the steam supply controls developed when the steam supply to the

1111 pump was automatically isolated by PDIS-1-17 and 18. Initial trouble-shooting could not

1 3 duplicate the problem. After three successful tests, the pump was declared operable

at 2300 CST. Subsequent investigations resulted in modifying the setpoints for

PDfS-1-17 and 18.

FACILITY STATUS	POWER	OTHER STATUS	METHOD OF DISCOVERY	DISCOVERY DESCRIPTION
		(30)		(32) Operator observation

ACTIVITY	CONTENT	AMOUNT OF ACTIVITY	LOCATION OF RELEASE
7	8 9	10 11	12 13
14	15 16	17 18	19 20
21	22 23	24 25	26 27
28	29 30	31 32	33 34
35	36 37	38 39	40 41
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63	64 65	66 67	68 69
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98	99 100	101 102	103 104
105	106 107	108 109	110 111
112	113 114	115 116	117 118
119	120 121	122 123	124 125
126	127 128	129 130	131 132
133	134 135	136 137	138 139
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210	211 212	213 214	215 216
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1 6 2 34 NA NA

PERSONNEL EXPOSURES		
NUMBER	TYPE	DESCRIPTION
1	1	1
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100	100	100

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PERSONNEL INJURIES

NUMBER				DESCRIPTION
1	8	0	0	0
			(40)	NA

7 8 9 10 11 12
LOSS OF OR DAMAGE TO FACILITY (43)
TYPE DESCRIPTION

1 9 2 47 NA

PUBLICITY ISSUED DESCRIPTION (45) NRC USE ONLY

7	8	9	10	NA	68	69
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LER SUPPLEMENTAL INFORMATION

SQRO-50-327/81024 Technical Specification Involved: 3.7.1.2.b

Reported Under Technical Specification: 6.9.1.13.b

Date of Occurrence: 3/14/81 Time of Occurrence: 1204 CST

Identification and Description of Occurrence:

Operator experienced difficulty with steam supply controls to steam-driven auxiliary feedwater pump 1A-S. Pump was declared inoperable.

Conditions Prior to Occurrence:

Unit in mode 3 following reactor trip

Apparent Cause of Occurrence:

An initial pressure spike (over 50-feet WC) in addition to a bias error caused by a clog in the sense lines to PDIS-1-17 and 18 (approximately 5-feet WC) caused PDIS-1-17 and 18 to isolate the steam supply to the AFW pump turbine.

Corrective Action:

The setpoints for PDIS-1-17 and 18 were temporarily modified with engineering design's concurrence from 55.68 ft. WC to 95 ft. WC to prevent recurrence of this event in the future. A permanent setpoint change will be implemented by a design change request now being prepared and expected to be submitted by May 1, 1981.