

UNITED STATES NUCLEAR REGULATORY COMMISSION WASHINGTON, D. C. 20555

March 30, 1981

Docket No. 50-334

Mr. C. N. Dunn, Vice President Operations Division Duquesne Light Company 435 Sixth Avenue Pittsburgh, Pennsylvania 15219



Dear Mr. Dunn:

We have reviewed your letter, dated March 28, 1981, in which you requested authorization to use Code Case N-210 with the conditions specified in Regulatory Guide 1.147 in the Beaver Valley Power Station Unit 1 Inservice Inspection Program. You also requested relief from the requirement of subsection IWA 5210, paragraph (a) of ASME Section XI 1974 Edition through Summer 1975 Addenda.

The use of Code Case N-210 with the conditions specified in Regulatory Guide 1.147 allows the substitution of an operational pressure test for a hydrostatic test when repair or modification is made to Reactor Coolant System (RCS) piping. This code case has been previously reviewed and accepted by the staff and its application to the Beaver Valley Power Station, Unit 1 Inservice Inspection Program (ISI) is acceptable.

Subsection IWA 5210, paragraph (a) of ASME Section XI 1974 Edition through Summer 1975 Addenda requires a four hour hold time at system test pressure prior to performing the required visual inspection. The use of a ten minute hold time prior to performing the usual inspection for uninsulated and exposed components or piping was adopted in the ASME Section XI 1977 Edition and is acceptable for use in the Beaver Valley Power Station, Unit 1 ISI Program.

We have determined that this relief does not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the relief involves an action which is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR \$51.5(d)(4), that an environmental impact appraisal need not be prepared in connection with the granting of this relief.

We have concluded, based on the considerations discussed above that: (1) because the relief does not involve a significant increase in the probability or consequences of accidents previously considered and some not involve a significant

hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the granting of this relief will not be inimical to the common defense and security or to the health and safety of the public.

Sincerely,

Steven A. Varga, Chief

Operating Reactors Branch No. 1

Division of Licensing

cc: See next page

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