



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

Jena

Docket No. 50-395

MAR 20 1981

Mr. T. C. Nichols, Jr.
Vice President & Group Executive
Nuclear Operations
South Carolina Electric & Gas Company
P. O. Box 764
Columbia, South Carolina 29281



Dear Mr. Nichols:

SUBJECT: REQUEST FOR ADDITIONAL INFORMATION

Our review of your application indicates that additional information is required to resolve open matters related to the Summer Preservice Inspection Program. The information that we require is identified in the enclosure.

We request that you provide the information not later than March 30, 1981. If you require any clarification of these matters, please contact the staff's assigned project manager.

Sincerely,

Robert L. Tedesco
Assistant Director for Licensing
Division of Licensing

Enclosure:
Request for Additional
Information

cc w/enclosure:
See next page

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Mr. T. C. Nichols, Jr.
Vice President & Group Executive
Nuclear Operations
South Carolina Electric & Gas Company
P. O. Box 764
Columbia, South Carolina 29218

MAR 20 1981

cc: Mr. William A. Williams, Jr.
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South Carolina Public Service Authority
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Resident Inspector/Summer NPS
c/o U. S. NRC
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Jenkinsville, South Carolina 29065

ENCLOSURE

V.C. SUMMER NUCLEAR STATION

121.0 Review of Preservice Inspection Program

121.30 To complete our evaluation of the applicant's Preservice Inspection Plan, Revision 1, we will require the specific information requested in Question 121.24, as follows:

A. Vessel Calibration Blocks

Page B-29 -- Confirm that the subject calibration blocks used to examine a clad component are clad in accordance with I-3122 of Section XI.

B. Class 1 Piping (Category B-J), Page B-32 and Class 2 Piping (Categories C-F and C-G), Page B-41.

Provide a list of the piping welds that cannot be completely examined to Section XI requirements. Estimate the extent of the Section XI requirement that was performed. State the primary reason that a specific examination is impractical, e.g., support or component restricts access, fitting prevents adequate ultrasonic coupling on one side, component to component weld prevents ultrasonic examination, etc.

C. Residual Heat Exchanger, Page B-36. Confirm that the nozzle-to-vessel welds were radiographed during fabrication.

D. Seal Water Heat Exchanger, Page B-37, Reactor Coolant Filtr Page B-38, Seal Water Return Filter, Page B-39, Letdown Re-Heat Heat Exchanger, Page B-40, Describe the examination method and extent of examination coverage during fabrication on these component welds.

121.31 Paragraph 50.55a(b)(2)(iv) requires that the Containment Heat Removal System shall be examined. The PSI Plan states in Section 1.2.4.a.5 (Page B-4) that the containment spray system is exempt from volumetric and surface examination based on IWC-1220(b) of Section XI because the system does not function during normal reactor operation.

Describe the measures or component modifications that would be required to performed a volumetric and/or surface examination of a representative sample of welds during the inservice inspection program.