

POOR ORIGINAL

OPERATING DATA REPORT

DOCKET NO. 50-321
DATE
COMPLETED BY
TELEPHONE (912) 367-7781 x 203

OPERATING STATUS

- 1 Unit Name: E. I. Hatch Nuclear Plant Unit 1
- 2 Reporting Period: 02-81
- 3 Licensed Thermal Power (MWT): 2436
- 4 Nameplate Rating (Gross MWe): 809.3
- 5 Design Electrical Rating (Net MWe): 777.3
- 6 Maximum Dependable Capacity (Gross MWe): 796.3
- 7 Maximum Dependable Capacity (Net MWe): 756.8
- 8 If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons:

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* Notes

* Cumulative totals reported in the January report have been found to be incorrect but have been corrected per conversation with Phil Ross of the NRC on 3-18-81. Totals shown this month are correct.

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- 9 Power Level To Which Restricted, If Any (Net MWe):
- 10 Reasons For Restrictions, If Any:

	This Month	Yr.-to-Date	Cumulative
11 Hours In Reporting Period	672	1416	45264
12 Number Of Hours Reactor Was Critical	564.3	1216	34728.4
13 Reactor Reserve Shutdown Hours	0.0	0	0.0
14 Hours Generator On-Line	546.9	1158.3	32603.7
15 Unit Reserve Shutdown Hours	0.0	0.0	0.0
16 Gross Thermal Energy Generated (MWH)	1030056	2156314	67959196
17 Gross Electrical Energy Generated (MWH)	337860	709680	22009330
18 Net Electrical Energy Generated (MWH)	320567	673160	20925638
19 Unit Service Factor	81.4	81.8	72.0
20 Unit Availability Factor	81.4	81.8	72.0
21 Unit Capacity Factor (Using MDC Net)	63.0	62.8	61.1
22 Unit Capacity Factor (Using DER Net)	61.4	61.2	59.5
23 Unit Forced Outage Rate	15.7	16.9	20.1
24 Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):			

25 If Shut Down At End Of Report Period, Estimated Date of Startup: June 1, 1981

26 Units In Test Status (Prior to Commercial Operation):	Forecast	Achieved
INITIAL CRITICALITY	-----	-----
INITIAL ELECTRICITY	-----	-----
COMMERCIAL OPERATION	-----	-----

(9777)

8108810464

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO 50-321
 DATE
 COMPLETED BY
 TELEPHONE (912) 367-7781 x 203

MONTH 02-61

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	618	17	477
2	545	18	615
3	621	19	618
4	623	20	618
5	624	21	611
6	620	22	619
7	613	23	620
8	621	24	622
9	620	25	623
10	615	26	623
11	251	27	499
12	435	28	-10
13	-13	29	
14	-12	30	
15	-10	31	
16	52		

(9/77)

POOR ORIGINAL

UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-321
 UNIT NAME Hatch 1
 DATE 3-10-81
 COMPLETED BY J. M. Kamishlian
 TELEPHONE 912-367-7851

REPORT MONTH February

No.	Date	Type ¹	Duration (Hours)	Reason ²	Method of Shutting Down Reactor ³	Licensee Event Report #	System Code ⁴	Component Code ⁵	Cause & Corrective Action to Prevent Recurrence
81-15	810201	S	27.3	F	5	NA	ZZ	ZZZZZ	Management limitation of 80% thermal power
81-16	810202	F	10.3	A	5	NA	CH	FILTER	Load reduced in order to reduce heat load on equipment caused by PSW strainer plugging
81-17	810202	F	6.7	B	5	NA	ZZ	CONROD	Load reduced to perform rod pattern adjustment
81-18	810202	S	5.5	F	5	NA	ZZ	ZZZZZ	Management limitation of 80% thermal power
81-19	810207	S	2	B	5	NA	HE	TURBIN	Load reduced to perform weekly turbine test

1
 F: Forced
 S: Scheduled

2
 Reason:
 A-Equipment Failure (Explain)
 B-Maintenance of Test
 C-Refueling
 D-Regulatory Restriction
 E-Operator Training & License Examination
 F-Administrative
 G-Operational Error (Explain)
 H-Other (Explain)

3
 Method:
 1-Manual Scram.
 2-Manual Scram.
 3-Automatic Scram.
 4-Continuations
 5-Load Reduction
 9-Other (Explain)

4
 Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

5
 Exhibit I - Same Source

UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-321
 UNIT NAME Hatch 1
 DATE 3-10-81
 COMPLETED BY J. M. Kamishlian
 TELEPHONE 912-367-7851

REPORT MONTH February

No.	Date	Type ¹	Duration (Hours)	Reason ²	Method of Shutting Down Reactor ³	Licensee Event Report #	System Code ⁴	Component Code ⁵	Cause & Corrective Action to Prevent Recurrence
81-20	810207	S	98.2	F	5	NA	ZZ	ZZZZZZ	Management limitation of 80% thermal power
81-21	810211	F	.1	A	5	NA	HA	INSTRU	Load reduced due to leakage of EHC oil in the #2 control valve
81-22	810211	F	10.9	A	2	NA	HA	INSTRU	Rx scram due to leakage of EHC oil in the #81-21
81-23	810211	F	25.3	A	5	NA	HA	INSTRU	Unit on startup ramp from above shutdown #81-22
81-24	810212	F	94.5	A	1	1-81-010	EE	SUPPORT	Unit brought to cold shutdown due to inadequate seismic supports for diesel generator PSW piping

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 B-Maintenance of Test
 C-Refueling
 D-Regulatory Restriction
 E-Operator Training & License Examination
 F-Administrative
 G-Operational Error (Explain)
 H-Other (Explain)

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UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH February

DOCKET NO. 50-321
 UNIT NAME Unit 1
 DATE 3-10-81
 COMPLETED BY J. M. Kamishlian
 TELEPHONE 12-367-781

No.	Date	Type ¹	Duration (Hours)	Reason ²	Method of Shutting Down Reactor ³	Licensee Event Report #	System Code ⁴	Component Code ⁵	Cause & Corrective Action to Prevent Recurrence
81-25	810216	F	23.6	A	5	NA	EE	SUPPORT	Unit on startup ramp from above shutdown #81-24
81-26	810217	F	9.4	A	5	NA	HH	HTEXCH	Load reduced due to loss of condenser vacuum
81-27	810218	S	71.7	F	5	NA	ZZ	ZZZZZZ	Management limitation of 80% thermal power
81-28	810220	S	2.0	B	5	NA	HE	TURBIN	Load reduced to perform weekly turbine test
81-29	810221	S	152.0	F	5	NA	ZZ	ZZZZZZ	Management limitation of 80% thermal power
81-20	810227	F	6.5	A	5	81-010	CB	PUMPXX	Recirc pumps tripped while performing HNP-1-3960

¹ F: Forced
S: Scheduled

² Reason:
A-Equipment Failure (Explain)
B-Maintenance of Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative
G-Operational Error (Explain)
H-Other (Explain)

³ Method:
1-Manual
2-Manual Scram.
3-Automatic Scram.
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5-Load Reduction
9-Other (Explain)

⁴ Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

⁵ Exhibit I - Same Source

(9/77)

UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-371
 UNIT NAME Hatch 1
 DATE 3-10-81
 COMPLETED BY J. M. Kamishlian
 TELEPHONE 912-367-7851

REPORT MONTH February

No.	Date	Type ¹	Duration (Hours)	Reason ²	Method of Shutting Down Reactor ³	Licensee Event Report #	System Code ⁴	Component Code ⁵	Cause & Corrective Action to Prevent Recurrence
81-31	810227	S	11.6	C	5	NA.	ZZ	ZZZZZZ	Unit shutting down for refueling outage
81-32	810228	S	20.3	C	3	NA	CD	VALVEX	Rx scram due to MSIV closure

¹
F: Forced
S: Scheduled

²
 Reason:
 A-Equipment Failure (Explain)
 B-Maintenance or Test
 C-Refueling
 D-Regulatory Restriction
 E-Operator Training & License Examination
 F-Administrative
 G-Operational Error (Explain)
 H-Other (Explain)

³
 Method:
 1-Manual
 2-Manual Scram.
 3-Automatic Scram.
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⁵
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NARRATIVE REPORT
UNIT 1

Feb. 1st Unit limited to 80% thermal power by management

Feb. 2nd Reduced load at 3:20 in order to reduce heat load on equipment due to plant service water strainers plugging

Feb. 2nd At 13:45 load was reduced to perform rod pattern adjustment

Feb. 2nd Unit limited to 80% thermal power by management

Feb. 7th Reduced load to perform weekly turbine test

Feb. 7th Unit limited to 80% thermal power by management

Feb. 11th At 4:11 load was reduced due to leakage of EHC oil at #2 control valve

Feb. 11th Rx manual scram at 4:20 due to leakage of EHC oil at #2 control valve

Feb. 11th Unit on startup ramp from above shutdown; Rx critical at 12:10; gen. on line at 15:13

Feb. 12th Unit going into cold shutdown at 16:30 due to inadequate seismic supports for diesel generator plant service water piping; main turbine offline at 19:43 and Rx mode switch in S/D at 4:20 on 810213

Feb. 16th Unit on startup ramp from above shutdown; Rx critical at 4:20; gen. on line at 15:03

Feb. 17th At 14:35 load reduced due to loss of condenser vacuum

Feb. 18th Unit limited to 80% thermal power by management

Feb. 20th Load reduced at 23:40 to perform weekly turbine test

Feb. 21st Unit limited to 80% thermal power by management

Feb. 27th

At 9:35 load was reduced due to recirc pumps tripping at 16:04 Rx power decreasing in order to shutdown for refueling

Feb. 28th

At 1:08 gen off line due to refueling outage; at 3:22 Rx auto scrambled due to MSIV closure

HATCH 1 SAFETY-RELATED MAINTENANCE REQUESTS
TO BE REPORTED FOR FEBRUARY 1981

<u>NUMBER</u>	<u>DATE COMPLETED</u>	<u>DESCRIPTION</u>
80-6659	12-30-80	Disassembled RCIC trip and throttle valve 1E51-C002-2, polished poppet, welded leak off line back in place and replaced motor and set valve switches
80-2671	1-5-81	Completed work on reactor building Ess MCCs R21-S001, S012 according to DCR 79-194