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POWER BUILDING  
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WILLIAM G. PARKER, JR.  
VICE PRESIDENT  
STEAM PRODUCTION

March 25, 1981

TELEPHONE AREA 704  
373-4083

Mr. Harold R. Denton, Director  
Office of Nuclear Reactor Regulation  
U.S. Nuclear Regulatory Commission  
Washington, D. C. 20555

Attention: Mr. B. J. Youngblood, Chief  
Licensing Projects Branch No. 1

Re: McGuire Nuclear Station  
Docket Nos. 50-369  
NUREG-0737



Dear Mr. Denton:

This letter is to confirm our telephone discussion of March 24, 1981 regarding Item II.F.2 as well as to provide Duke Power Company's position on several other outstanding review items on McGuire.

II.F.2

Prior to June 1, 1981, Duke Power will replace the existing analog meter on the core exit thermocouple monitoring panel with an installed display capable of reading to at least 2300°F.

A loss of power to the reference junction thermocouple enclosure will not preclude utilization of the installed readout capability.

Duke Power is evaluating the core exit thermocouple system in light of the NRC Staff's review guidance, R.G. 1.97 and Attachment 1 to II.F.2 and will provide its assessment of the system by April 23, 1981.

With regard to the Reactor Vessel Level Indicating System (RVLIS) testing program, it is our understanding that the "developmental" testing is being conducted jointly by Westinghouse, NRC and EG&G personnel. Upon completion of this testing, a report documenting the results would presumably be written. If necessary Duke Power will submit this report on the McGuire docket to support use of the RVLIS. Additionally, the results of the plant calibration and testing will be provided upon completion of these activities.

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I.D.2

This item requires each applicant to install a Safety Parameter Display System in accordance with NUREG 0696. NUREG 0696 was recently transmitted to Duke Power Company by letter of March 5, 1981. This letter stated that "The Commission has approved NUREG 0696 and noted that this document provides general guidance only, ... and that compliance with NUREG 0696 is not a requirement."

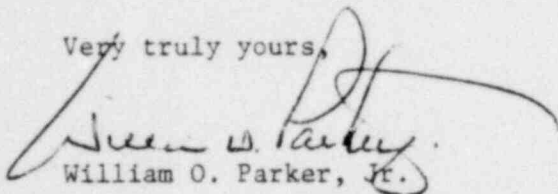
Additionally, NUREG 0700 is scheduled to be issued in the near future as guidance for conducting control room reviews. Duke will address the SPDS concept as part of its control room review. This is considered an appropriate course of action since any decision relating to safety displays must consider the integration of these displays with overall operating objectives.

II.D.1

Duke Power will provide verification of block valve functionability under all fluid conditions expected during operating and accident conditions by July 1, 1982. The basis for this verification will be functional testing with appropriate supporting analysis. A program outline describing the methods will be submitted by July 1, 1981.

Please advise if you have additional questions regarding any of these matters.

Very truly yours,



William O. Parker, Jr.

GAC:pw

cc: T. J. Donat  
Senior Resident Inspector  
McGuire Nuclear Station

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