

OPERATING DATA REPORT

DOCKET NO. 50-289  
 DATE 3-15-81  
 COMPLETED BY D. G. Mitchell  
 TELEPHONE (717) 948-8553

OPERATING STATUS

1. Unit Name: Three Mile Island Nuclear Station, Unit I
2. Reporting Period: February 1981
3. Licensed Thermal Power (MWt): 2535
4. Nameplate Rating (Gross MWe): 871
5. Design Electrical Rating (Net MWe): 819
6. Maximum Dependable Capacity (Gross MWe): 840
7. Maximum Dependable Capacity (Net MWe): 776
8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons:

9. Power Level To Which Restricted, If Any (Net MWe): \_\_\_\_\_
10. Reasons For Restrictions, If Any: \_\_\_\_\_

	This Month	Yr.-to-Date	Cumulative
11. Hours In Reporting Period	672.	1416.	56929.
12. Number Of Hours Reactor Was Critical	0.0	0.0	31731.8
13. Reactor Reserve Shutdown Hours	0.0	0.0	839.5
14. Hours Generator On-Line	0.0	0.0	31180.9
15. Unit Reserve Shutdown Hours	0.0	0.0	0.0
16. Gross Thermal Energy Generated (MWH)	0.0	0.0	76531071.
17. Gross Electrical Energy Generated (MWH)	0.	0.	25484330.
18. Net Electrical Energy Generated (MWH)	0.	0.	23840053.
19. Unit Service Factor	0.0	0.0	54.8
20. Unit Availability Factor	0.0	0.0	54.8
21. Unit Capacity Factor (Using MDC Net)	0.0	0.0	53.3
22. Unit Capacity Factor (Using DER Net)	0.0	0.0	51.1
23. Unit Forced Outage Rate	100.0	100.0	37.1

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):

25. If Shut Down At End Of Report Period, Estimated Date of Startup: \_\_\_\_\_

26. Units In Test Status (Prior to Commercial Operation):	Forecast	Achieved
INITIAL CRITICALITY	_____	_____
INITIAL ELECTRICITY	_____	_____
COMMERCIAL OPERATION	_____	_____

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-289

UNIT TMI-I

DATE March 15, 1981

COMPLETED BY D. G. Mitchell

TELEPHONE (717) 948-8553

MONTH February 1981

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	<u>0</u>
2	<u>0</u>
3	<u>0</u>
4	<u>0</u>
5	<u>0</u>
6	<u>0</u>
7	<u>0</u>
8	<u>0</u>
9	<u>0</u>
10	<u>0</u>
11	<u>0</u>
12	<u>0</u>
13	<u>0</u>
14	<u>0</u>
15	<u>0</u>
16	<u>0</u>

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
17	<u>0</u>
18	<u>0</u>
19	<u>0</u>
20	<u>0</u>
21	<u>0</u>
22	<u>0</u>
23	<u>0</u>
24	<u>0</u>
25	<u>0</u>
26	<u>0</u>
27	<u>0</u>
28	<u>0</u>
29	<u>0</u>
30	<u>0</u>
31	<u>0</u>

**UNIT SHUTDOWNS AND POWER REDUCTIONS**

REPORT MONTH February

DOCKET NO. 50-289  
 UNIT NAME TMI-1  
 DATE March 15, 1980  
 COMPLETED BY D. G. MJ. chell  
 TELEPHONE (717) 948-8553

No.	Date	Type <sup>1</sup>	Duration (Hours)	Reason <sup>2</sup>	Method of Shutting Down Reactor <sup>3</sup>	Licensee Event Report #	System Code <sup>4</sup>	Component Code <sup>5</sup>	Cause & Corrective Action to Prevent Recurrence
1	2/1/81	F	672	D	1				Regulatory Restraint Order

<sup>1</sup>  
 F: Forced  
 S: Scheduled

<sup>2</sup>  
 Reason:  
 A-Equipment Failure (Explain)  
 B-Maintenance or Test  
 C-Refueling  
 D-Regulatory Restriction  
 E-Operator Training & License Examination  
 F-Administrative  
 G-Operational Error (Explain)  
 H-Other (Explain)

<sup>3</sup>  
 Method:  
 1-Manual  
 2-Manual Scram.  
 3-Automatic Scram.  
 4-Other (Explain)

<sup>4</sup>  
 Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

<sup>5</sup>  
 Exhibit I - Same Source

## OPERATIONS SUMMARY

The Unit was shutdown the entire month by order of the NRC. Core Cooling was provided by the Decay Heat Removal System.

### MAJOR SAFETY RELATED MAINTENANCE

During the month of February, the Unit was in cold shutdown while restart activities were in progress. The following major maintenance items were performed.

Refueling Outage Surveillance Testing continued for the snubber program with snubbers being tested and reinstalled, minor bleed rate adjustments being performed. As required by Technical Specifications, additional snubbers were pulled as a result of failures. This program will continue into March.

The local leak rate testing surveillance programs, started in February, progressed with no excessive leak rates recorded. This surveillance will continue in March.

Replacement of one of the Industrial Coolant Isolation Valves (RB-V-7) was in progress during this month with the following items being performed.

1. Insulation removed from valve and pipe around valve.
2. Pneumatic operator removed.
3. Valve cut from pipe.
4. Weld preparations performed.
5. Fitup inspections performed.
6. Hot/root pass welds performed.
7. Final weld out in progress.

This work was performed with satisfactory results.

Restart task on PORV and Safety Valve Indication (tie-in portion), was performed with satisfactory results. This included welding of nipples to elbows, drilling of holes into elbow, installation of pipe to first isolation valve. No problems were encountered while performing this work.

REFUELING INFORMATION REQUEST

1. Name of Facility:

Three Mile Island Nuclear Station, Unit I

2. Scheduled date for next refueling shutdown:

Unknown

3. Scheduled date for restart following refueling:

Unknown

4. Will refueling or resumption of operation hereafter require a technical specification change or other license amendment?

If answer is yes, in general, what will these be?

If answer is no, has the reload fuel design and core configuration been reviewed by your Plant Safety Review Committee to determine whether any unreviewed safety questions are associated with the core reload (Ref. 10 CFR Section 50.59)?

If no such review has taken place, when is it scheduled?

Amendment No. 50, Cycle 5 reload, was approved on 3-16-79.

5. Scheduled date (s) for submitting proposed licensing action and supporting information:

N/A

6. Important licensing considerations associated with refueling, e.g. new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures:

N/A

7. The number of fuel assemblies (a) in the core, and (b) in the spent fuel storage pool:

(a) 177

(b) 208

8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned, in number of fuel assemblies:

The present licensed capacity is 752. There are no planned increases at this time.

9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity:

1986 is the last refueling discharge which allows full core off-load capacity (177 fuel assemblies).