50-155

C. J. Hartman Plant Superintendent

Big Rock Point Nuclear Plant, Box 591, Route 3, Charlevoix, Michigan 49720

February 4, 1981

Office of Management Information & Process Control Records Management Branch United States Nuclear Regulatory Commission Washington, DC 20555

Gentlemen:

Attached, please find the statistical data for the Big Rock Point Nuclear Plant covering the period January 1 through January 31, 1981.

Sincerely,

C: Mantinters

C J Hartman Plant Superintendent

CJH:nh

cc: Region III - Directorate of Regulatory Operations DEVanFarowe - Department of Public Health Ron Callen - Michigan Public Service Commission Resident Inspector - Nuclear Regulatory Commission - Site R/C - RBDeWitt - P26-117B DPHoffman - P24-115 JJDellas - P11-228B RLRosenfeld - P24-402 LPhillips - P21-327 Document Control Center - BRP - 740*22*10*04 File

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attachments

GREYBOOK OPERATING DATA REPORT

		DOCKET NO. 50-155	DATE:	2/2/81
1.2.3.	OPERATING STATUS UNIT NAME: BIG ROCK POINT NUCLEAR PLANT REPORTING PERIOD: 1 / 81 LICENSED THERMAL POWER (MWT): 240	NOTES: Nearing end of refueling outage.	BY : PHONE :	S.AMSTUTZ 616-547-6537,EX 180
5. 6. 7. 8.	DESIGN ELECTRICAL RATING (GROSS MWE): 75 MAXIMUM DEPENDABLE CAPACITY (GROSS MWE): 68.0 MAXIMUM DEPENDABLE CAPACITY (NET MWE): 64.0 IF CHANGES OCCUR IN CAPACITY RATINGS(ITEMS 3 THRU 7) SINCE LAST REPORT, GIVE (REASONS :	
9.	POWER LEVEL TO WHICH DESTRICTED TO ANY WET WHICH			

EL TO WHICH RESTRICTED, IF ANY (NET MWE): 53.0 0. REASONS FOR RESTRICTIONS, IF ANY:

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	THIS MONTH	YEAR-TO-DATE	CUMULATIVE
 HOURS IN REPORTING PERIOD NUMBER OF HOURS REACTOR WAS CRITICAL REACTOR RESERVE SHUTDOWN HOURS HOURS GENERATOR ON-LINE UNIT RESERVE SHUTDOWN HOURS GROSS THERMAL ENERGY GENERATED (MWH) GROSS ELECTRICAL ENERGY GENERATED (MWH) NET ELECTRICAL ENERGY GENERATED (MWH) UNIT SERVICE FACTOR UNIT AVAILABILITY FACTOR UNIT CAPACITY FACTOR (USING MDC NET) UNIT CAPACITY FACTOR (USING DER NET) 	744.0 10.9 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0	744.0 10.9 0.0 0.0 0.0 0.0 0.0 0.0 0.0 0.0 2.0 2.0	156451.0 107104.0 0.0 104930.8 0.0 19521201.0 6165089.0 5833789.9 67.12 67.12 55.12
3. UNIT FORCED OUTAGE RATE	0.0%	0.0%	22.3%

4. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS(TYPE, DATE, & DURATION OF EACH):

. IF SHUT DOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP: February 4, 1981

>				
, <u>u</u>	AVERAGE DAILY	POWER(MWT) (NWEN)	(1/81)	
•	0.0	0.0		1
	2 0.0	0.0		
	3 0.0	0.0		1. 1. 1. 1. 1. 1. 1. 1. 1. 1. 1. 1. 1. 1
	4 0.0	0.0		
	5 0.0	0.0		
•	6 0.0	0.0		
	7 0.0	0.0		
	8 0.0	0.0		1.
	0.0	0.0		
	. 0.0	0.0		
	2 0.0	0.0		2 Funder
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1	5 0.0	0.0		
• 1	6 0.0	0.0		
1	7 0.0	0.0		
1	8 0.0	0.0		e-ste
1	9 6.0	0.0		234 <u>4</u>
- 2	0.0	C.0		1.22
• 2	1 0.0	0.0		100
2	2 0.0	0.0		
2	3 0.0	0.0		1.20
• 2	4 0.0	0.0		19.22
2	5	0.0		
4	0.0	0.0		
•	0.0	0.0		
	0 0.0	0.0		
7	0.0	0.0		
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AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. <u>50-155</u> UNIT <u>Big Rock Point Plar</u> DATE <u>02/04/81</u> COMPLETED BY <u>SAmstutz</u> TELEPHONE <u>616-547-6537</u> x 180



INSTRUCTIONS

On this format, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

(9/77)

50-155 DOCKET NO. UNIT SHUTDOWNS AND POWER REDUCTIONS Big Rock Point Plant UNIT NAME February 4, 1981 DATE COMPLETED BY .SAmstutz REPORT MONTH January TELEPHONE 616-547-6537 x 180 Method of Shutting Down Reactor³ Component Code5 Reason? Duration (Hours) System Code⁴ Cause & Corrective Licensee Typel Action to Event Date No. Prevent Recurrence Report # 3 NONE < -3 4 2 Exhibit G - Instructions Method: F: Forced Reason: for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-I-Manual A-Equipment Failure (Explain) B-Maintenance or Test S: Scheduled 2-Manual Scram. 3-Automatic Scram. C-Refueling D-Regulatory Restriction E-Operator Training & License Examination F-Administrative 4-Other (Explain) 0161) 5 Exhibit I - Same Source G-Operational Error (Explain) II-Other (Explain) (9/77)

UNIT SHUTDOWNS AND POWER REDUCTIONS

INSTRUCTIONS

This report should describe all plant shutdowns during the report period. In addition, it should be the source of explanation of significant dips in average power levels. Each significant reduction in power level (greater than 20% reduction in average daily power level for the preceding 24 hours) should be noted, even though the unit may not have been shut down completely¹. For such reductions in power level, the duration should be listed as zero, the method of reduction should be listed as 4 (Other), and the Cause and Corrective Action to Prevent Recurrence column should explain. The Cause and Corrective Action to Prevent Recurrence column should be used to provide any needed explanation to fully describe the circumstances of the outage or power reduction.

NUMBER. This column should indicate the sequential number assigned to each shutdown or significant reduction in power for that calendar year. When a shutdown or significant power reduction begins in one report period and ends in another, an entry should be made for both report periods to be sure all shutdowns or significant power reductions are reported. Until a unit has achieved its first power generation, no number should be assigned to each entry.

DATE. This column should indicate the date of the start of each shutdown or significant power reduction. Report as year, month, and day. August 14, 1977 would be reported as 770814. When a shutdown or significant power reduction begins in one report period and ends in another, an entry should be .nade for both report periods to be sure all shutdowns or significant power reductions are reported.

TYPE. Use "F" or "S" to indicate either "Forced" or "Scheduled," respectively, for each shutdown or significant power reduction. Forced shutdowns include those required to be initiated by no later than the weekend following discovery of an off-normal condition. It is recognized that some judgment is required in categorizing shutdowns in this way. In general, a forced shutdown is one that would not have been completed in the absence of the condition for which corrective action was taken.

DURATION. Self-explanatory. When a shutdown extends beyond the end of a report period, count only the time to the end of the report period and pick up the ensuing down time in the following report periods. Report duration of outages rounded to the nearest tenth of an hour to facilitate summation. The sum of the total outage hours plus the hours the generator was on line should equal the gross hours in the reporting period.

REASON. Categorize by letter designation in accordance with the table appearing on the report form. If category H must be used, supply brief comments.

METHOD OF SHUTTING DOWN THE REACTOR OR REDUCING POWER. Categorize by number designation in accordance with the table appearing on the report form. If category 4 must be used, supply brief comments.

LICENSEE EVENT REPORT #. Reference the applicable reportable occurrence pertaining to the outage or power reduction. Enter the first four parts (event year, sequential report number, occurrence code and report type) of the five part designation as described in Item 17 of Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161). This information may not be immediately evident for all such shutdowns, of course, since further investigation may be required to ascertain whether or not a reportable occurrence was involved.) If the outage or power reduction will not result in a reportable occurrence, the positive indication of this lack of correlation should be noted as not applicable (N/A).

SYSTEM CODE. The system in which the outage or power reduction originated should be noted by the two digit code of Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161).

Systems that do not fit any existing code should be designated XX. The code ZZ should be used for those events where a system is not applicable.

COMPONENT CODE. Select the most appropriate component from Exhibit I - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161), using the following critieria:

- A. If a component failed, use the component directly involved.
- B. If not a component failure, use the related component: e.g., wrong valve operated through error; list valve as component.
- C. If a chain of failures occurs, the first component to malfunction should be listed. The sequence of events, including the other components which fail, should be described under the Cause and Corrective Action to Prevent Recurrence column.

Components that do not fit any existing code should be designated XXXXXX. The code ZZZZZZ should be used for events where a component designation is not applicable.

CAUSE & CORRECTIVE ACTION TO PREVENT RECUR-RENCE. Use the column in a narrative fashion to amplify or explain the circumstances of the shutdown or power reduction. The column should include the specific cause for each shutdown or significant power reduction and the immediate and contemplated long term corrective action taken, if appropriate. This column should also be used for a description of the major safety-related corrective maintenance performed during the outage or power reduction including an identification of the critical path activity and a report of any single release of radioactivity or single radiation exposure specifically associated with the outage which accounts for more than 10 percenof the allowable annual values.

For long textual reports continue narrative on separate paper and reference the shutdown or power reduction for this narrative.

Note that this differs from the Edison Electric Institute (EEI) definitions of "Forced Partial Outage" and "Scheduled Partial Outage," For these terms, EEI uses a change of 30 MW as the break point. For larger power reactors, 30 MW is too small a change to warrant explanation.

Refueling Information Request

 Facility name: Big Rock Point Plant

1 1 5 5

- Scheduled date for next refueling shutdown: December, 1981
- Scheduled date for restart following shutdown: February, 1982
- 4. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment? No

If yes, explain. '

If no, has the reload fuel design and core configuration been reviewed by Plant Safety Review Committee to determine whether any unreviewed safety questions are associated with the core reload (Ref.10CFR,Sec.50.59)?

No If no review has taken place, when is it scheduled? August, 1981

- Scheduled date(s) for submittal of proposed licensing action and supporting information:
 - None
- 6. Important licensing considerations associated with refueling, 2.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures:
- 7. Number of fuel assemblies in: core 84 ; spent fuel storage pool 108

8. Present licensed spent fuel storage capacity: 193

Size of any increase in licensed storage capacity that has been requested or is planned (in number of fuel assemblies): 248

 Projected date of the last cafueling that can be discharged to spent fuel pool assuming the present licensed capacity: