

Doc# No. 50-346

License No. NPF-3

Serial No. 678

January 15, 1981



RICHARD P. CROUSE
Vice President
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Director of Nuclear Reactor Regulation
Attention: Darrell G. Eisenhut, Director
Division of Licensing
United States Nuclear Regulatory Commission
Washington, D.C. 20555

Dear Mr. Eisenhut:

On January 5, 1981, Toledo Edison submitted (letter Serial No. 675) a package to support de-automation of Incident Level 5 of the Safety Features Actuation System (SFAS) at the Davis-Besse Nuclear Power Station, Unit 1, (DB-1). Also on that day, members of my staff met with you and your staff in Bethesda, Maryland, to discuss this proposal for manual action to provide appropriate long-term post-loss of coolant accident (LOCA) core cooling.

Several discussions have transpired since that time with your staff. As a result, we have altered the proposed modification compared to the one presented on January 5, 1981. Toledo Edison's letter of January 9, 1981, (Serial No. 676) transmitted the electrical drawings describing the current proposed electrical changes. Specifically, the proposed modification retains the SFAS Incident Level 5 to provide a permissive function for control room operator action. This letter transmits the following:

Attachment A: Three (3) original and forty (40) conformed copies of an application for an Amendment to the Davis-Besse Facility Operating License No. NPF-3. This attachment supersedes in total Attachment A of Toledo Edison's January 5, 1981, submittal. This Attachment A identifies the specific proposed changes to the technical specifications, the safety evaluation supporting analysis and the schedule to implement the proposed amendment.

Attachment B: This provides the draft proposed modifications to the emergency operating procedures and the system operating procedures that are to be made reflecting the actions required to manually initiate long-term post-LOCA recirculation core cooling. Only the major revisions from Attachment B of the January 5, 1981, submittal are provided. All previous references to six feet will now read eight feet.

Attachment C: This provides, in verbal form, a description of how the electrical modification proposed in our submittal of January 9, 1981, functions.

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All control room personnel will be trained in the procedural revisions prior to their standing watches after system modification. Additionally, the prime responsibility for ensuring the manual shifting will be assigned to a dedicated licensed operator. This person is planned to be the same operator currently stationed for reactor coolant pump termination upon injection mode initiation.

This physical modification requires testing to verify its completion. As a result, this modification is to be completed during an outage period. This will be done the first outage of duration greater than 72 hours in operational Mode 4 after NRC approval.

Very truly yours,

R. P. Crouse / wgl

RPC:TJM:aa

cc: DB-1 NRC Resident Inspector

Docket No. 50-346
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Attachment A

APPLICATION FOR AMENDMENT

TO

FACILITY OPERATING LICENSE NO. NPF-3

DAVIS-BESSE NUCLEAR POWER STATION

UNIT I

Enclosed are forty (40) copies of the requested changes to the Davis-Besse Nuclear Power Station Unit No. 1 Facility Operating License No. NPF-3, together with the supporting Safety Evaluation and Analysis for the requested change. This requested change deals with the proposal to de-automate the valve shifts caused by Incident Level 5 of the Safety Features Actuation System.

The proposed Technical Specifications changes include pages:

3/4 1-17, 3/4 3-13, 3/4 3-48, 3/4 3-50, 3/4 5-3, 3/4 5-6
3/4 5-7, 3/4 6-11 and B3/4-12.

By W. Crowles
Director, Nuclear Services

For R. P. Crouse
Vice President, Nuclear

Sworn and subscribed before me this 15th day of January, 1981.

Judith Hirsch
Notary Public