

**LICENSEE EVENT REPORT**

CONTROL BLOCK:

(PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

1	2	3	4	5	6	7	8	9	10	11	12	13	14	15	16	17	18	19	20	21	22	23	24	25	26	27	28	29	30	31	32	33	34	35	36	37	38	39	40	41	42	43	44	45	46	47	48	49	50	51	52	53	54	55	56	57	58	59	60	61	62	63	64	65	66	67	68	69	70	71	72	73	74	75	76	77	78	79	80	81	82	83	84	85	86	87	88	89	90	91	92	93	94	95	96	97	98	99	100
N C B E P														0 0 - 0 0 0 0 0 - 0 0											4 1 1 1 1																																																																										
LICENSEE CODE														LICENSE NUMBER											LICENSE TYPE											CAT SR																																																															
CONT										REPORT SOURCE										L 0 5 0 - 0 3 2 4											1 2 1 2 8 0											0 1 0 8 8 1																																																									
																				DOCKET NUMBER											EVENT DATE											REPORT DATE																																																									

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

02 | During a normal reactor startup, control rod 26-35 had to be bypassed in the

03 | "full-out" position to allow the startup to continue. A second qualified person was

0 4 | present to verify compliance with the prescribed rod pattern. This event did not

05 | affect the health or safety of the public.

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08 Technical Specifications 3.1.4.2b, 6.9.1.9b

SYSTEM CODE 0 9		CAUSE CODE I B (11)		CAUSE SUBCODE X (12)		COMPONENT CODE I N S T R U (14)				COMP SUBCODE X (15)		VALVE SUBCODE Z (16)	
7 8		9 10		11		12 13		13 18		19		20	
(17) LER NO REPORT NUMBER		EVENT YEAR 8 0		SEQUENTIAL REPORT NO. 1 0 2		OCCURRENCE CODE 0 3		REPORT TYPE L		REVISION NO. 0			
21 22		23		24 26		27		28 29		30		31 32	
ACTION TAKEN X (18)		FUTURE ACTION X (19)		EFFECT ON PLANT Z (20)		SHUTDOWN METHOD Z (21)		HOURS 0 0 0 7 (22)		ATTACHMENT SUBMITTED N (23)		NPRD-4 FORM SUB. Y (24)	
33 34		35		36		37 40		41		42		43	
PRIME COMP SUPPLIER N (25)		COMPONENT MANUFACTURER G 0 8 0 (26)											
44		45 46 47											

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

1	0	This event is believed to have occurred due to a faulty signal from the rod position
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reed switch to the RSCS system. This problem will be fully investigated and resolved.

1 2 | during the next outage of sufficient length that allows personnel access to the dry-

1 3 | well. Until the problem is resolved this rod will be periodically checked for position

1 4 | during normal operation and reactor startups.

FACILITY STATUS (30) % POWER (29) OTHER STATUS (30) METHOD OF DISCOVERY (31) DISCOVERY DESCRIPTION (32)

1 5 8 9 C 23 10 0 0 0 12 13 NA 44 45 A 31 46 Operational Event 80

ACTIVITY CONTENT RELEASED OR RELEASE (33) AMOUNT OF ACTIVITY (35) LOCATION OF RELEASE (36)

1 6 8 9 Z 33 10 Z 34 11 NA 44 45 NA 46

PERSONNEL EXPOSURES											
NUMBER		TYPE		DESCRIPTION (39)							
1	7	0	0	0	(37)	2	(38)	NA			
7	8	9	10	11	12	13					

PERSONNEL INJURIES		DESCRIPTION
NUMBER		
1	2	NA

7 8 9 10 11 12

40 41

DOOD ORIGINAL

1		2		3		4		5		6		7		8		9		10		11		12		13		14		15		16		17		18		19		20		21		22		23		24		25		26		27		28		29		30		31		32		33		34		35		36		37		38		39		40		41		42		43		44		45		46		47		48		49		50		51		52		53		54		55		56		57		58		59		60		61		62		63		64		65		66		67		68		69		70		71		72		73		74		75		76		77		78		79		80		81		82		83		84		85		86		87		88		89		90		91		92		93		94		95		96		97		98		99		100	
1		2		3		4		5		6		7		8		9		10		11		12		13		14		15		16		17		18		19		20		21		22		23		24		25		26		27		28		29		30		31		32		33		34		35		36		37		38		39		40		41		42		43		44		45		46		47		48		49		50		51		52		53		54		55		56		57		58		59		60		61		62		63		64		65		66		67		68		69		70		71		72		73		74		75		76		77		78		79		80		81		82		83		84		85		86		87		88		89		90		91		92		93		94		95		96		97		98		99		100	

PUBLCITY										NRC USE ONLY									
ISSUED		DESCRIPTION		45		NA		68		69		80							
2	0	N	44																

NAME OF PREPARER M. J. Pastva, Jr.

PHONE 919-457-9521

NRC USE ONLY

1-800-851-9226