



Consumers
Power
Company

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October 11, 1974

Mr. Angelo Giambusso
Deputy Director for Reactor Projects
Directorate of Licensing
US Atomic Energy Commission
Washington, DC 20545

Re: Docket 50-155
License DPR-6
Big Rock Point Plant
ATWS

Dear Mr. Giambusso:

Consumers Power Company (CP Co) stated in our October 1, 1974 letter to you that a report concerning Anticipated Transients Without Scram (ATWS) for the Big Rock Point Plant would be delayed in submission beyond the October 1, 1974 due date. This report, as required by WASH-1270, "Technical Report on Anticipated Transients Without Scram for Water-Cooled Power Reactors," would be in two parts. The first part would be a review of the reactor shutdown system with a consideration of common mode failures. The second part would be an analysis of ATWS consequences.

The first part of the ATWS Report is a document entitled, "Big Rock Point Reactor Protection System Common Mode Failure Analysis," NEDC-20640. This document was prepared by General Electric Company (GE) and was received on September 27, 1974. After review by CP Co, a meeting was held with GE on October 3, 1974 to revise and update the report. Copies of the revised report will be transmitted to you in a few days.

With regards to possible ATWS consequences analysis, GE has informed us that the earliest possible date by which such analyses could be completed is April 1, 1976. GE pleads that manpower and resources are limited to handle its customer-related licensing issues. Specifically on the basis of potential impact on plant operation and public safety issues, GE has ranked ATWS last in priority behind ECCS - Appendix K models, fuel densification, scram reactivity and GETAB. GE has done no work on ATWS consequence analysis for Big Rock Point and will not be able to start this work for several months.

Accordingly, CP Co requests an extension until April 1, 1976 for submittal of any ATWS consequences analysis. It does not appear feasible at this time to have a more rapid analysis done by another consultant.

The consequences of the worst ATWS appear to be within the design pressure of the primary coolant boundary. The steam drum safety relief valves were sized to limit the pressure resulting from a coincident steam shutoff with failure to scram (see Sections 12.5.8, 12.12.6, and Figure

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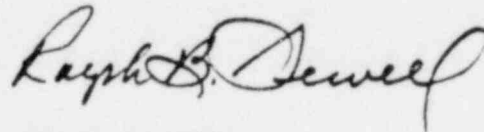
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12.10 of the Final Hazards Summary Report for the Big Rock Point Plant). Starting from operating conditions of 1500 psia and 240 Mwt, the peak pressure during this transient ranges from 1700 to 1800 psia. The design pressure of the reactor vessel is 1715 psia and the normal operating pressure is 1350 psia, instead of 1500 psia. Taking credit for the 150 psi lower operating pressure (1500-1350) would indicate peak transient pressures in the range of 1550 to 1650 psia, which is less than the reactor vessel design pressure.

Yours very truly,



Ralph B. Sewell
Nuclear Licensing Administrator

TWC/map

CC: JGKepler,
USAEC