



UNITED STATES
ATOMIC ENERGY COMMISSION
DIRECTORATE OF REGULATORY OPERATIONS
REGION III
799 ROOSEVELT ROAD
GLEN ELLYN, ILLINOIS 60137

TELEPHONE
(312) 858-2660

December 27, 1972

Consumers Power Company
ATTN: Mr. Robert L. Haueter
Electric Production
Superintendent - Nuclear
212 West Michigan Avenue
Jackson, Michigan 49201

Docket No. 50-155
Docket No. 50-255

Gentlemen:

Thank you for your letter of November 9, 1972, in response to our letter dated September 13, 1972, and your supplemental letter, dated December 6, 1972, which you have provided to clarify certain questionable aspects of your November 9 letter.

We have reviewed both of the above identified letters as they relate to the adequacy of the program you plan to implement to assure that certain valves in your Big Rock Point and Palisades Plants meet wall thickness requirements. Our review of your planned valve wall thickness verification program has been based on the guidelines contained in our letter of June 29, 1972, and a second letter, dated September 13, 1972, and we comment as follows:

With respect to your plans for the Palisades Plant, you apparently intend to determine the adequacy of valve wall thicknesses on a sampling basis and have identified a sample size of about 30 percent, insofar as the total number of valves is concerned, and a minimum sample size of about 12 percent relative to types of valves and manufacturers. While a sampling plan may be an acceptable approach, the method of sample selection, sample size, acceptance levels, and analysis of results of measurement activities become critical factors. Consequently, please provide us with additional information, in writing, within 30 days, which relates to: (1) the probability of failure of the sampling plan to identify a thin wall valve, and (2) the level of confidence involved.

Concerning your comments relative to the Big Rock Point Plant, i.e., the last two paragraphs of your November 9 letter, it is necessary that we receive additional information, in writing, within 30 days, in order to implement our evaluation of your position in this matter. This information should include, but

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not be limited to: (1) a listing of the valves involved based on the contents of our June 29, and September 13, 1972, letters, (2) identification of the design pressure rating and highest anticipated operating pressure for each of these valves, and (3) an analysis of valve design pressure rating versus highest anticipated operating pressure for each valve which can be evaluated in terms of the maximum amount of reduced wall thickness that could be accommodated by design overpressure considerations.

Your cooperation with us in this matter is appreciated.

Sincerely yours,

Boyce H. Grier
Regional Director

cc: Ralph B. Sewell, Nuclear
Licensing Administrator

bcc w/ltrs dtd 11/9/72 and 12/6/72:
RO Chief, RT&OB
RO Chief, RCB
RO:HQ (4)
Licensing (4)
DR Central Files
PDR
Local PDR
NSIC
DTIE
GC, Beth