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November 1, 1972

Regulatory

File Cy.

Mr. John F. O'Leary, Director Directorate of Licensing US Atomic Energy Commission Washington, DC 20545 Re: Docket No 50-155
License No DPR-6
Additional Information Proposed Tech Spec
Change No 31

Dear Mr. O'Leary:

Transmitted herewith are 40 copies of a report titled "Small and Intermediate Break Loss-of-Coolant Analysis for the Big Rock Point Reactor With ADS and JNC Reload G Fuel." This report is submitted to provide the results of analyses performed for small and intermediate break sizes for the loss-of-coolant accident for Reload G fuel. Design break accident results are as described in Proposed Technical Specifications Change No 31 dated June 16, 1972.

The analysis performed utilizes the blowdown data (including assumptions that an automatic depressurization system is installed) and heat transfer coefficients presented in our September 22, 1972 submittal titled "Big Rock Point Loss-of-Coolant Analysis With Automatic Depressurization and NFS Demonstration Fuel." Rod heatup calculations were performed by Jersey Nuclear Company using the MOXY code, as they were for the Design Basis Accident described in Proposed Technical Specifications Change No 31.

Yours very truly,

RBS/map

Ralph B. Sewell

Nuclear Licensing Administrator

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CC: BHGrier, USAEC



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