

Regulatory Docket File



Consumers
Power
Company

General Offices: 212 West Michigan Avenue, Jackson, Michigan 49201 • Area Code 517 788-0550

May 11, 1976

Director of Nuclear Reactor Regulation
US Nuclear Regulatory Commission
Washington, DC 20555



DOCKET 50-155, LICENSE DPR-6 -
BIG ROCK POINT PLANT

On April 19, 1976 you recommended to the commission in your "Comments by the Director, Nuclear Reactor Regulation, Relating to the Request for Exemption of the Big Rock Point Nuclear Power Plant From the Requirements of 10 CFR Section 50.46" that five items (Page 25) be completed by Consumers Power Company prior to the return of the Big Rock Point Plant to service from the present outage. The purpose of this letter is to address our actions with respect to these items.

Three of these items (1a, 1b and 1d) were addressed in our proposed Technical Specifications dated May 10, 1974. A fourth item (1c) is discussed in this letter and additional information is provided on one item discussed in the May 10, 1976 submittal. Information regarding the fifth and remaining item (1e) will be submitted by the end of this week.

Item 1c states:

"Modify the emergency procedures to assure a second emergency diesel will be obtained and operational within 24 hours after a LOCA."

The emergency procedures will be modified prior to start-up to require that, following a LOCA, a chain of events be initiated that will result in delivery of a diesel generator to the site such that it can be made fully operational within 24 hours after a LOCA.

A suitable trailer-mounted diesel generator is located at our Marysville Plant and arrangements have been made for Big Rock Point priority use in the event of a LOCA. Arrangements have also been made through a nearby Division office for the use of a Consumers Power Company tractor to deliver the diesel generator to the Big Rock Point Plant. A test will be conducted prior to start-up to ensure that the diesel generator can be operational within 24 hours following a LOCA.

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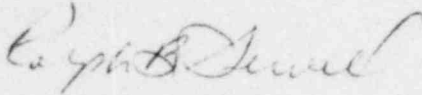
Item 1b recommended that augmented surveillance be provided for the core ring spray system. Our May 10, 1976 proposed Technical Specifications included enhanced component operability surveillance. In addition, since April 19, 1976, we have examined all accessible welds in the Class I portion of the core ring spray system.

The examination of the Class I portion was performed following the requirements of Sections III and XI of the ASME B&PV Code, 1971 Edition with Addenda through winter 1972. The following table shows the items examined and method of examination: (See attached isometric for item locations.)

<u>Item</u>	<u>Class 6</u>	<u>Section XI Req Exams</u>	<u>Exams Performed</u>	<u>Remarks</u>
Weld #1	II	None	UT	
Weld #2	II	None	UT	
Weld #3	II	None	UT	
Weld #4	I	UT	UT	
Weld #5	I	UT	UT	
Weld #6	I	UT	UT	
Weld #7	I	UT	UT	
Weld #8	I	UT	UT	
Support Lug CSH-8	I	UT, VT	UT, VT	
Weld #9	I	UT	UT	
Hanger CSH-7A	I	VT	VT	
Hanger CSH-7	I	VT	VT	
Weld #10	I	UT	UT	
Weld #11	I	UT	UT	
Weld #12	I	UT	UT	
Weld #13	I	UT	UT	
Hanger CSH-6	I	VT	VT	
Weld #14	I	UT	UT	
Weld #14A	I	UT	UT	
Weld #15	I	UT	UT	
Weld #16	I	UT	UT	
Hanger CSH-5	I	VT	VT	
Weld #17	I	UT	UT	
Weld #17A	I	UT	UT	
Weld #18	I	UT, PT, VT	UT, PT, VT	Dissimilar Metal Weld
Weld #19	I	UT	None	Not Accessible
Weld #20	I	UT	None	Not Accessible
Weld #21	I	UT	None	Not Accessible
Weld #22	I	UT	None	Not Accessible

The above examinations have been performed by Southwest Research Institute and Consumers Power Laboratory Services. The appropriate documentation shall be included in the respective 1976 Inservice Inspection reports for these groups.

No indications were found volumetrically. One indication was found with penetrant when examining Weld #18. This indication was an arc-strike and was subsequently ground out and reexamined and found to be acceptable.



Ralph B. Sewell
Nuclear Licensing Administrator

CC: JGKepler,
USNRC

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Ralph B. Sewell

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Pt. Plant....

PLANT NAME: Big Rock Pt. Plant

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ACKNOWLEDGED

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FOR ACTION/INFORMATION

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