

Regulatory Docket File



Consumers  
Power  
Company

General Offices: 212 West Michigan Avenue, Jackson, Michigan 49201 • Area Code 517 788-0550

May 10, 1976

Director of Nuclear Reactor Regulation  
US Nuclear Regulatory Commission  
Washington, DC 20555



DOCKET 50-155, LICENSE DPR-6 -  
BIG ROCK POINT PLANT

Transmitted herewith are three (3) executed and thirty-seven (37) conformed copies of a request for a change to the Technical Specifications of License DPR-6, Docket 50-155, issued to Consumers Power Company on May 1, 1964 for the Big Rock Point Plant.

This proposed change provides enhanced operability and surveillance requirements for the core spray systems, modifies the automatic actuation requirements of the containment spray systems such that the interlock between MO-7064 and MO-7068 can be eliminated, and changes the allowable operating pressure to be consistent with the ECCS analysis submitted July 25, 1975.

Also attached is an analysis of ECCS performance for the case of a break in the ring core spray line assuming only reflooding from the nozzle spray system and feed-water system. This analysis shows that the core is never uncovered.

Ralph B. Sewell  
Nuclear Licensing Administrator

CC: JGKepler, USNRC

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Case No. / Ltr. Dated 5-10-76

CONSUMERS POWER COMPANY

DOCKET 50-155

REQUEST FOR CHANGE TO THE TECHNICAL SPECIFICATIONS

BIG ROCK POINT PLANT

For the reasons hereinafter set forth, the following changes to the Technical Specifications of License No DPR-6 issued to Consumers Power Company on May 1, 1964 for the Big Rock Point Plant are requested:

I. CHANGES

A. Delete Section 3.5.2(a) in its entirety.

B. Delete Section 3.5.2(c) in its entirety.

C. Delete the following phrases from Section 4.1.2(b):

" , core spray and backup core spray systems"

and

"core spray system"

and

"and the fire water makeup system to the condenser hotwell"

and

"core spray system and"

and

"and the breakers for M07070 and M07071 shall be tagged 'open'"

D. Delete Section 4.2.1(a) in its entirety.

E. Delete Section 4.2.1(b) in its entirety.

F. Delete the last sentence in Section 4.2.6.

G. In the table contained in Section 5.2.1(b), change the entry for the row now labeled "Maximum Reactor Pressure During Power Operation, Psig" to read as follows:

"Nominal Reactor Pressure During Steady State Power Operation, Psig"	1335	1335
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H. Delete Section 6.1.4(a) in its entirety.

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- I. Delete Section 6.1.4(b) in its entirety.
- J. Delete Section 6.1.4(c) in its entirety.
- K. Change Section 6.1.5(b) to read as follows:  

"The emergency condenser system control initiation sensors shall be functionally tested not less frequently than once every 12 months."
- L. Delete Section 6.1.6 in its entirety.
- M. Delete the entries corresponding to "Post-Incident Spray System - Automatic Operation" in the table contained in Section 7.6.
- N. Change the words "Reactor Emergency Core Cooling System Trip Circuits" in the table in Section 7.6 to "Emergency Condenser Trip Circuits."
- O. Add a new Section 11.3.1.4 as follows: