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OCT 27 1980

MEMORANDUM FOR: W. Kreger, Assistant Director  
for Radiation Protection  
Division of Systems Integration

FROM: G.C. Lainas, Assistant Director  
for Safety Assessment  
Division of Licensing

SUBJECT: USE OF A TWO ASSEMBLY FUEL SIPPING DEVICE

The purpose of this memorandum is to assure that you are aware of activities at operating reactors, relating to radiation protection concerns. The attached memorandum from the Office of Inspection and Enforcement deals with the subject of using a fuel assembly sipping device to sip two assemblies at the same time. IE has determined that utility's safety evaluation performed under 10 CFR 50.59 was appropriate. We agree with the IE determination for this case. However, it would be more desirable to have situations like this addressed in the FSAR, if such situations can be anticipated, since this would eliminate the need for a licensee review (under 10 CFR 50.59) and IE or NRR reviews as the situations develop. Therefore, it may be worthwhile to consider modifying the Standard Review Plan (Section 15.7.4) to address such cases.

We will attempt to keep you informed of operating reactor events and activities relating to radiation protection.

Original signed by

*D. CRUTCHFIELD*

G.C. Lainas, Assistant Director  
for Safety Assessment  
Division of Licensing

*for*

Contact:  
G. Holahan, X27112

Attachment:  
As stated

cc w/attachment:  
G. Lainas  
J. Olshinski  
G. Holahan  
L. Barrett  
K. Wichman

8012220

524

*RD-3  
Fuel  
Assembly*

|         |             |            |            |  |
|---------|-------------|------------|------------|--|
| OFFICE  | DL:ORAB/SL  | DL:ORAB/BC | DL:ORAB/SL |  |
| SURNAME | GHolahan:sh | JOlshinski | GLainas    |  |
| DATE    | 10/21/80    | 10/21/80   | 10/22/80   |  |



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

SSINS: 6530

OCT 9 1980

MEMORANDUM FOR: G. C. Lainas, Assistant Director for Safety Assessment,  
DL, NRR

FROM: Leo B. Higginbotham, Assistant Director, Division of  
Fuel Facility and Materials Safety Inspection, IE

SUBJECT: FUEL HANDLING ACCIDENT - USE OF A TWO ASSEMBLY FUEL  
SIPPING DEVICE

Enclosed for your consideration is a 10 CFR 50.59 safety evaluation performed by the Sacramento Municipal Utility District (Rancho Seco facility). The operation being supported by the evaluation is the use of a two assembly fuel sipping device as opposed to a single assembly sipper. IE has reviewed the licensee's evaluation and has agreed with the licensee's determination that an unreviewed safety question does not exist when considering the operational and time restrictions placed on the use of the sipper.

The licensee's evaluation is based on the failure of two complete fuel assemblies. This two assembly failure is outside the bounds on the routine NRR fuel handling accident consequences, which considers the failure of only one fuel assembly. Therefore, we are providing a copy of the licensee's evaluation for NRR's generic consideration of fuel handling accidents and the potential for more than a single effective fuel assembly failure.

Leo B. Higginbotham  
Assistant Director  
Division of Fuel Facility and  
Materials Safety Inspection, IE

Enclosure: As Stated

cc: J. H. Sniezek, IE  
W. Kreger, NRR  
S. Bryan, IE  
W. Houston, DSE

Dupe of 8412160278

1. DESCRIPTION: Shipping of as many as two Spent Fuel Assemblies at a time will be accomplished in the Spent Fuel Pool Cask Loading Pit. Special test equipment will be provided by B&W; the installation and operation of this equipment is covered by STP 224

David P. Whitman Date 12-3-  
Cognizant Operating Engineer

2. TEST OR EXPERIMENT: CALCULATIONS  SAFETY ANALYSIS   
Calculations and Safety Analysis to support this operation are attached.

David P. Whitman Date 12-3-79 W.D. Beach Date 12-  
Cognizant Engineer Manager Gen. Engr.

3. PRC RECOMMENDATION: 50.59(a) Yes  No  50.59(b) Yes  No

DISPOSITION OF PRC:  
a. Unanimously recommends proposal   
b. Send to MSRC for concurrence   
c. Recommends not to proceed   
d. Safety Analysis inadequate   
e. MSRC review prior to implementing   
Rowan Colomo Date 12-11-  
PRC Chairman

4. ANALYSIS: 50.59(a) Yes  No  Recommend to Proceed: Yes  No  Refer to MSRC

50.59(b) Yes  No   
[Signature] Date 2-28-  
Plant Superintendent

5. MSRC FINDINGS: 50.59(a) Yes  No  50.59(b) Yes  No

DISPOSITION OF MSRC:  
a. Recommends proposal   
b. Send to NRC for approval   
c. Recommends not to proceed   
d. Safety analysis inadequate   
[Signature] Date 4/19/80

6. COMMISSION APPROVAL OBTAINED NA THIS IS 59b  
Date MSRC Chairman

7. WORK DISPOSITION: Test or experiment completed.  
[Signature] Date 7/21/80  
Cognizant Engineer

8. OVERALL REVIEW: Documentation complete. Test or experiment complete.

POOR ORIGINAL

Manager Nuclear Operations Date Dupe of 8012160282  
Quality Assurance Director Date