U.S. NUCLEAR REGULATORY COMMISSION

LICENSEE EVENT REPORT

CONTROL BLOCK / / / / / / / (1) (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

/0/1/ /V/A/N/A/S/1/ (2) /0/0/-/0/0/0/0/0/-/0/0/ (3) /4/1/1/1/1 (4) / / / (5)

LICENSEE CODE LICENSE NUMBER LICENSE TYPE CAT

/0/1/ REPORT SOURCE /L/ (6) /0/5/0/0/0/3/3/8/ (7) /1/1/1/4/8/0/ (8) /1/2/0/3/8/0/ (9)

DOCKET NUMBER EVENT DATE REPORT DATE

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

/0/2/ / On November 14, 1980, while operating in Mode 1, LI-1474 indicated greater than /

/0/3/ / 5% higher than the average of the three channels, thereby providing a non- /

/0/4/ / conservative reading in the event of a low steam generator level signal. In /

/0/5/ / addition, this non-conservative reading was logged previously and the channel /

/0/6/ / not declared inoperable. Since there were two redundant channels available /

/0/7/ / to trip the reactor in the event of a low steam generator level, the health /

/0/8/ / and safety of the public were not affected. /

SYSTEM CODE CAUSE CODE CAUSE SUBCODE COMPONENT CODE COMP. SUBCODE VALVE SUBCODE

/0/9/ /I/A/ (11) /E/ (12) /E/ (13) /I/N/S/T/R/U/ (14) /T/ (15) /Z/ (16)

LER/RO REPORT NUMBER (17) EVENT YEAR SEQUENTIAL REPORT NO. OCCURRENCE CODE REPORT TYPE REVISION NO.

/8/0/ /-/ /0/9/7/ / \ / /0/3/ /L/ /-/ /0/

ACTION TAKEN FUTURE ACTION EFFECT ON PLANT SHUTDOWN METHOD HOURS ATTACHMENT SUBMITTED NPRD-4 FORM SUB. PRIME COMP. SUPPLIER COMPONENT MANUFACTURER

/B/ (18) /Z/ (19) /Z/ (20) /Z/ (21) /0/0/0/0/ (22) /Y/ (23) /N/ (24) /A/ (25) /W/1/2/0/ (26)

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

/1/0/ / The cause of this event was high transmitter output due to a leaking root /

/1/1/ / isolation valve. The corrective action was to furmanite the leaking isolation /

/1/2/ / valve and perform a channel check on the affected channel. The check was /

/1/3/ / successful, and the channel placed back in service. /

/1/4/ / /

FACILITY STATUS %POWER OTHER STATUS METHOD OF DISCOVERY DISCOVERY DESCRIPTION (32)

/1/5/ /E/ (28) /0/9/0/ (29) / NA / (30) /B/ (31) / Operator Observation /

ACTIVITY RELEASED CONTENT OF RELEASE AMOUNT OF ACTIVITY (35) LOCATION OF RELEASE (36)

/1/6/ /Z/ (33) /Z/ (34) / NA / / NA /

PERSONNEL EXPOSURES

NUMBER TYPE DESCRIPTION (39)

/1/7/ /0/0/0/ (37) /Z/ (38) / NA /

PERSONNEL INJURIES

NUMBER DESCRIPTION (41)

/1/8/ /0/0/0/ (40) / NA /

LOSS OF OR DAMAGE TO FACILITY (43)

TYPE DESCRIPTION

/1/9/ /Z/ (42) / NA /

PUBLICITY

ISSUED DESCRIPTION (45) NRC USE ONLY

/2/0/ /N/ (44) / NA / / / / / / / / / / / /

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Virginia Electric and Power Company
North Anna Power Station, Unit 1 Attachment: Page 1 of 1
Docker No. 50-338
Report No. LER 80-097/03L-0

Description of Event

On November 14, 1980 while operating in Mode 1, it was discovered that LI-1474, Steam Generator Level Channel 1 - Protection, was indicating >5% higher than the average of the three channels, thereby providing a non-conservative reading in the event of a low steam generator level. In addition, the non-conservative channel was logged twice previously without the affected channel being declared inoperable and placed in trip within one hour. This event is applicable to T.S. 3.3.1.1 and reportable pursuant to T.S. 6.9.1.9.a,c.

Probable Consequences of Occurrence

Since there were two redundant channels available to trip the reactor in the event of a low steam generator level, the health and safety of the public were not affected.

Cause of Event

1-FW-LT-1474, Steam Generator Level Channel 1 - Protection, indicated 5% greater than the average of the three channels due to a transmitter with excessive high output caused by a steam leak on the root isolation valve.

Immediate Corrective Action

Within one hour after discovering that the affected channel had been out of spec for the last 8 hours, a channel calibration check was performed and the affected channel declared inoperable, and placed in trip. Subsequent to being placed in trip, the transmitter was found to have an abnormally high output due to a steam leak on a root isolation valve. The transmitter was isolated and the valve furmanited to seal the leak. The channel was returned to operation and a comparative channel check was successfully performed. The affected channel was then declared operable and returned to service.

Scheduled Corrective Action

There was no scheduled corrective action.

Actions Taken to Prevent Reccurrence

All personnel involved in the incident were counseled on the importance of carefully reviewing data from all logs for acceptability. No further corrective action was required.

Generic Implications

There are no generic implications.