



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

YANKEE ATOMIC ELECTRIC COMPANY

DOCKET NO. 50-29

YANKEE NUCLEAR POWER STATION (YANKEE-ROWE)

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 42
License No. DPR-3

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Yankee Atomic Electric Company (the licensee) dated August 2, 1977, as supplemented August 9, 1977, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-3 is hereby amended to read as follows:

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"(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 42, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications."

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

J. V. Wambach

for

A. Schwencer, Chief
Operating Reactors Branch #1
Division of Operating Reactors

Attachment:
Changes to the Technical
Specifications

Date of Issuance: August 19, 1977

ATTACHMENT TO LICENSE AMENDMENT NO. 42

FACILITY LICENSE NO. DPR-3

DOCKET NO. 50-29

Revise Appendix A as follows:

Remove the following pages and insert identically numbered pages.

3/4 4-25 / 3/4 4-26
3/4 4-27 / 3/4 4-28

Marginal lines indicate revised area. Overleaf pages are provided for convenience.

MAIN COOLANT SYSTEM

PRESSURIZER

LIMITING CONDITION FOR OPERATION

3.4.8.2 The pressurizer temperature shall be:

- a. Limited to a maximum heatup or cooldown of 200°F in any one hour period,
- b. Within 225°F of the Main Coolant System temperature, and
- c. Greater than 70°F whenever pressurizer pressure exceeds 500 psig.

APPLICABILITY: At all times.

ACTION:

With the pressurizer temperature outside of any of the above limits, restore the temperature to within the limits within 30 minutes; perform an engineering evaluation to determine the effects of the out-of-limit condition on the fracture toughness properties of the pressurizer; determine that the pressurizer remains acceptable for continued operation or be in at least HOT STANDBY within the next 6 hours and reduce the pressurizer pressure to less than 500 psig within the following 30 hours.

SURVEILLANCE REQUIREMENTS

4.4.8.2 The pressurizer temperatures shall be determined to be within the limits at least once per 30 minutes during system heatup or cooldown. The spray water temperature differential shall be determined to be within the limit at least once per 12 hours during manual spray operation.

MAIN COOLANT SYSTEM

STRUCTURAL INTEGRITY

CLASS I COMPONENTS

LIMITING CONDITION FOR OPERATION

3.4.9 The structural integrity of Main Coolant System components (except steam generator tubes) identified in Table 4.4-3 as Class 1 components shall be maintained at a level consistent with the acceptance criteria in Specifications 4.4.9.1, 4.4.9.2, 4.4.9.3 and 4.4.9.4.

APPLICABILITY: MODES 1, 2, 3 and 4.

ACTION:

With the structural integrity of any of the above components not conforming to the above requirements, restore the structural integrity of the affected component to within its limit or isolate the affected component prior to increasing the Main Coolant System temperature more than 50°F above the minimum temperature required by NDT considerations. The provisions of Specification 3.0.4 are not applicable.

SURVEILLANCE REQUIREMENTS

4.4.9.1 The following inspection program shall be performed during shutdown:

- a. Inservice Inspections The structural integrity of the Class 1 components shall be demonstrated by verifying their acceptability when inspected per the applicable requirements of Section XI of the ASME Boiler and Pressure Vessel Code, 1970 Edition, and Addenda through Winter 1970, as outlined by the inspection program shown in Table 4.4-3.

For all Class 1 piping the ultrasonic calibration shall be per:

1. Article III-200 of Appendix III - ASME Sec XI-Summer 1976 Addenda except that III-2410 shall be deleted, III-2430 shall be used except 50% Reference level recording shall be performed. Ten percent overlap shall be retained.
2. Article III-3000 shall be used entirely.
3. Article III-4000 shall be used entirely.
4. Supplement 7 shall be used for austenitic welds.

MAIN COOLANT SYSTEM

SURVEILLANCE REQUIREMENTS (Continued)

An initial report of any abnormal degradation of the structural integrity of the Safety Class 1 components detected during the above required inspections shall be made within 10 days after detection and the detailed report shall be submitted pursuant to Specification 6.9.4 within 90 days after completion of the surveillance requirements of this specification.

The Inservice Inspection Program shall be reviewed every 5 years to assure that the equipment, techniques and procedures being utilized are current and applicable. The results of these reviews shall be reported in Special Reports to the Commission pursuant to Specification 6.9.6 within 90 days of completion.

- b. Inspections Following Repairs or Replacements The structural integrity of the Main Coolant System shall be demonstrated after completion of all repairs and/or replacements to the system by verifying the repairs and/or replacements meet the requirements of Section XI of the ASME Boiler and Pressure Vessel Code, 1970 Edition, and Addenda through Winter 1970, except for all Class 1 piping the ultrasonic calibration shall be per:

1. Article III-2000 of Appendix III - ASME Sec XI - Summer 1976 except that III-2410 shall be deleted, III-2430 shall be used except 50% Reference level recording shall be performed. Ten percent overlap shall be retained.
2. Article III-3000 shall be used entirely.
3. Article III-4000 shall be used entirely.
4. Supplement 7 shall be used for austenitic welds.

When repairs and/or replacements are made which involve new strength welds on components greater than 2 inch diameter, the new welds shall receive a surface and 100 percent volumetric examination and meet applicable code requirements. When repairs and/or replacements are made which involve new strength welds on components 2 inch diameter or smaller, the new welds shall receive a surface examination and meet applicable code requirements.

MAIN COOLANT SYSTEM

SURVEILLANCE REQUIREMENTS (Continued)

- c. Inspections Following System Opening The structural integrity of the Main Coolant System shall be demonstrated after each closing by performing a leak test, with the system pressurized to at least 2200 psig, in accordance with Section XI of the ASME Boiler and Pressure Vessel Code, 1970 Edition, and Addenda through Winter 1970, and the Pressure/Temperature limits of Specification 3.4.8.1.

4.4.9.2 The following inspection program shall be performed at least once per 18 months during shutdown on at least one shroud tube per quadrant .

- a. Inspect the integrity of the bolts and locking devices in the lower flange at the bottom of the shroud tubes.
- b. Inspect the interface between the shroud tube lower flange and the tie plate for separation.
- c. Inspect the interface between the shroud tube upper flange and the top shroud tube support plate for separation.
- d. Inspect the interface between the top shroud tube support plate and the lower core support plate for separation.
- e. Inspect for abnormalities one of each of the types of bolts per quadrant.

4.4.9.3 The pressurizer interior shall be inspected at least once per 18 months during shutdown using the best available techniques to determine if any change has occurred in the cladding cracks that exist and whether any further cracking of the cladding has taken place.

4.4.9.4 The 2" charging line between CH-MOV-524 and CH-611A shall be dye penetrant tested at least once per 18 months during shutdown.