SECTION 13

CONDUCT OF OPERATION

TABLE OF CONTENTS

Section	<u>Title</u>	<u>Page</u>
13.1	ORGANIZATION STRUCTURE	13.1-1
13.1.1	Deleted	13.1-1
13.1.1.1	Deleted	13.1-1
13.1.1.2	Deleted	13.1-1
13.1.1.3	Deleted	13.1-1
13.1.1.4	Deleted	13.1-1
13.1.1.4.1	Deleted	13.1-2
13.1.1.4.1.1	Deleted	13.1-1
13.1.1.4.1.2	Deleted	13.1-1
13.1.1.4.1.3	Deleted	13.1-1
13.1.1.4.1.4	Deleted	13.1-1
13.1.1.4.2	Deleted	13.1-1
13.1.1.4.3	Deleted	13.1-2
13.1.1.4.4	Radiation Protection	13.1-2
13.1.2	Deleted	13.1-2
13.1.3	Deleted	13.1-2
13.1.4	Deleted	13.1-2
13.1.5	Deleted	13.1-2
13.1.6	Deleted	13.1-2
13.1.7	Deleted	13.1-2
13.1.8	Deleted	13.1-2
13.1.9	Deleted	13.1-2
13.1.10	Deleted	13.1-2
13.1.11	Deleted	13.1-2
13.2	TRAINING	13.2-1
13.3	EMERGENCY PLANNING	13.3-1
13.4	REVIEW AND AUDIT	13.4-1
13.5	PLANT PROCEDURES	13.5-1
13.5.1	Administrative Procedures	13.5-2
13.5.2	Operating and Maintenance Procedures	13.5-3
13.5.3	Additional Operating and Maintenance Procedures	13.5-4
13.5.4	Technical Requirements Manual	13.5-7
13.6	PLANT RECORDS	13.6-1
13.7	SECURITY	13.7-1

TABLE OF CONTENTS (Cont)

LIST OF TABLES

<u>Table</u>	<u>Title</u>
13.1-1	Deleted
13.5-1	Administrative Procedure Topics
13.5-2	Abnormal Operating Procedures

TABLE OF CONTENTS (Cont)

LIST OF FIGURES

<u>Figure</u>	<u>Title</u>	
13.1-1	Deleted	
13.1-2	Deleted	
13.1-3	Deleted	
13.1-4	Deleted	

13-iii

SGS-UFSAR

SECTION 13

CONDUCT OF OPERATIONS

13.1 ORGANIZATION STRUCTURE

The original content of this chapter has been modified as allowed by Regulatory Guide 1.181 in conjunction with NEI 98-03, Guidelines for Updating Final Safety Analysis Reports (see UFSAR Appendix 3A).

On August 21, 2000, the operating licenses for the Salem Units 1 & 2, and for the Hope Creek station were transferred from Public Service Electric & Gas (PSE&G) to PSEG Nuclear LLC. PSEG Nuclear LLC, a limited liability company, is a subsidiary of Public Service Enterprise Group (PSEG), an investor-owned company headquartered in the State of New Jersey. PSEG Nuclear LLC is dedicated to the safe, reliable and efficient operation of the nuclear units and assumes full responsibility for meeting all license obligations. The PSE&G corporate organization and its functions and responsibilities are described in Chapter 2 of the Quality Assurance Topical Report NO-AA-10, as revised.

For the Hope Creek project, Bechtel Power Corporation and Bechtel Construction, Inc. designed and constructed the plant. General Electric Company designed, supplied and provided engineering support for the Nuclear Steam Supply System (NSSS) for the Hope Creek project. For the Salem projects, PSE&G and Westinghouse Electric Corporation jointly participated in the design and construction of each unit. PSE&G provided an experienced and trained staff to support preoperational testing, core load and power ascension testing programs of the nuclear units.

The PSEG Nuclear LLC organizational structure and reporting relationships are described in the Quality Assurance Topical Report NO-AA-10. Roles and responsibilities are described in administrative procedures.

- 13.1.1 Deleted
- 13.1.1.1 Deleted
- 13.1.1.2 Deleted
- 13.1.1.3 Deleted
- 13.1.1.4 Deleted
- 13.1.1.4.1 Deleted
- 13.1.1.4.1.1 Deleted
- 13.1.1.4.1.2 Deleted
- 13.1.1.4.1.3 Deleted
- 13.1.1.4.1.4 Deleted
- 13.1.1.4.2 Deleted

13.1.1.4.3 Deleted

13.1.1.4.4 Radiation Protection

The Radiation Protection Program and organization are described in Section 12.3.

- 13.1.2 Deleted
- 13.1.3 Deleted
- 13.1.4 Deleted
- 13.1.5 Deleted
- 13.1.6 Deleted
- 13.1.7 Deleted
- 13.1.8 Deleted
- 13.1.9 Deleted
- 13.1.10 Deleted
- 13.1.11 Deleted

13.1-2

SGS-UFSAR Revision 31
December 5, 2019

This Table Has Been Deleted

See HR-AA-104

1 of 1

SGS-UFSAR

Revision 31 December 5, 2019 **This Figure Has Been Deleted** PSEG NUCLEAR L.L.C. SALEM GENERATING STATJON RELATIONSHIP WITH PUBLIC SERVICE ENTERPRISE GROUP SHEET 1 OF 1 SALEM UFSAR- REV 31 December 5, 2019 F13.1-1

This Figure Has Been Deleted

PSEG NUCLEAR L.L.C.
SALEM GENERATING STATION

NUCLEAR ORGANIZATION

SALEM UFSAR - REV 31

SHEET 1 OF 1

December 5, 2019

F13.1-2

This Figure Has Been Deleted

PSEG NUCLEAR L.L.C. SALEM GENERATING STATION

SITE OPERATIONS ORGANIZATION

SALEM UFSAR - REV 31 SHEET 1 OF 1

December 5, 2019 F13.1-3

This Figure Has Been Deleted

PSEG NUCLEAR L.L.C. SALEM GENERATING STATION

STATION OPERATIONS DEPARTMENT

SALEM UFSAR - REV 31

SHEET 1 OF 1

December 5, 2019

F13.1-4

13.2 TRAINING

The training program is formulated to provide an organization qualified to operate, maintain and support the facilities in a safe and reliable manner. The training program has been developed from a systematic analysis of job requirements using job and task analysis where available. This approach is consistent with Nuclear Regulatory Commission (NRC) regulations and the Institute of Nuclear Power Operations (INPO) recommendations for accreditation of training programs.

13.2.1 Accredited Training Programs

The following training programs have been accredited by INPO and are based on a Systems Approach to Training (SAT):

*Shift Supervisor

Senior Reactor Operator

*Shift Technical Advisor

Reactor Operator

Nuclear Equipment Operator

Continuing Training for Licensed Personnel

*Nonlicensed Operator

Nuclear Technician - Chemistry

*Chemistry Technician

Nuclear Technician - Controls

*Instrument and Control Technician

*Electrical Maintenance Personnel

Nuclear Technician - Mechanical

*Mechanical Maintenance Personnel

Nuclear Technician - Radiation Protection

- *Radiological Protection Technician
- *Engineering Support Personnel

The contents and details of these programs are contained in the Nuclear Training Manual.

13.2.2 Supervisory Training Program

In addition to the technical training received by department personnel, the supervisory training program offers technical and management skills training for supervision. This training follows guidelines given in applicable INPO documents.

^{*}Titles referenced in 10CFR50.120

13.2.3 Site Access Training

In order to be granted unescorted access to the site protected area, all personnel (including temporary maintenance and service personnel) are required to successfully complete General Employee Training (GET) and Fitness For Duty (FFD) training (10CFR26). Supervisors are required to successfully complete the supervisory portion of the FFD training. GET covers the following areas:

- General site personnel response to emergency situations
- Basic radiation protection practices
- Overview of Quality Assurance/Assessment
- Industrial safety
- Security Program requirements
- Hazardous materials

In order to be granted unescorted access to radiation and/or contaminated areas, personnel are required to successfully complete the Radiation Worker Training (RWT) program.

In order to maintain unescorted access to the site protected areas and the radiation and/or contaminated areas, personnel are required to complete the appropriate retraining program for GET, FFD and RWT. The frequency of retraining for GET and RWT is determined by following the Systematic Approach to Training (SAT) based training process. Retraining for FFD is required on an annual basis.

Program/course plans for GET, RWT, and FFD are maintained with their associated training program documents.

13.2.4 Fire Brigade Training Program

Fire Brigade training is described in Section 9.5 and the Nuclear Training Manual.

13.3 EMERGENCY PLANNING

The Emergency Plan Manuals as revised describe the PSEG Nuclear LLC Emergency Preparedness Program. These manuals have been submitted as separate documents and provide a complete description of the emergency response for the Hope Creek and Salem Stations.

13.4 REVIEW AND AUDIT

The independent review and audit functions are performed by the Plant Operations Review Committee (PORC), the Nuclear Oversight (NOS) organization, and the Nuclear Safety Review Board (NSRB). See the Salem and Hope Creek Quality Assurance Topical Report (QATR) for additional details.

13.5 PLANT PROCEDURES

Plant procedures are prepared by the plant staff, support organizations or contract organizations for applicable activities delineated in the Technical Specifications, Section 6.8, at a minimum, and provide the controls necessary to comply with applicable Regulatory Guides as listed in Section 1.8 for Hope Creek and Appendix 3A for Salem. Procedures are prepared using a standard format and content, and a writer's guide which provides human factors and style guidance.

A technical review and control process utilizing qualified reviewers functions to perform periodic or routine review of procedures. When periodic review is used as the assessment method, these controls will establish a schedule for review. Processing Of Procedures and T&RMS, (AD-AA-101) and Writers Guide And Process Guide For Procedures and T&RMS, (AD-AA-101-1002) describe the review and approval process for procedures. Procedure changes that require a 10CFR50.59 evaluation are reviewed by the Plant Operations Review Committee (PORC).

Procedures are periodically reviewed and revised when changes are necessary or desirable. Similarly, procedures are reviewed and revised when necessary following the completion of system design changes or equipment modifications. Subsequent to an unexpected plant transient, significant operator error, etc.,

the appropriate procedure(s) receive a review. The purpose of this review is to ascertain whether the procedure may have contributed to the cause of the condition or was adequate in its capacity to mitigate the consequences.

Routine procedures provide the fundamental written guidance for routinely managing, operating and maintaining the plant. Routine plant procedures are assessed by users before and during use to determine if changes are necessary or desirable. Routine procedures receive an appropriate degree of scrutiny by individuals knowledgeable in the procedures, and are updated as necessary to ensure adequacy during suitably controlled activities such as normal procedure usage, development of plant modifications, industry experience reviews, licensing actions, training activities, corrective actions for nonconforming conditions, and quality assurance audits and surveillances.

Non-routine procedures are those procedures whose use is event-driven, such as Emergency Operating Procedures, Emergency Plan Implementing Procedures, Abnormal Procedures, and Alarm Response Procedures; these procedures will be reviewed every two years. However, if a non-routine procedure is fully exercised and there is a detailed scrutiny of the entire procedure as part of a documented training program, this may serve as the biennial review of the procedure.

13.5-1

Circumstances may develop during maintenance, operations and testing of plant systems when an existing procedure cannot be performed as written. The following prescribes the mechanisms which address these circumstances:

The On-The-Spot-Change (OTSC) Process may be used to change the content of implementing procedures provided that:

- The intent of the original procedure is not altered;
- The change is approved by two members of the unit's management staff, at least one of whom holds a Senior Reactor Operator's license holder on the unit affected and
- The change is documented and receives the same level of review and approval as the original procedure within 14 days of implementation.

The Partial Procedure Implementation Process may be used to omit procedure step(s) or section(s) (which may not be applicable or appropriate at the time of implementation) of implementing procedures provided that:

- The intent of the original procedure is not altered;
- The appropriateness of omitting the affected procedure step(s) or section(s) is reviewed by at least two members of the unit's management staff familiar with the procedure. However, for Maintenance implementing procedures, the appropriateness of omitting the affected procedure step(s) or section(s) may be reviewed by one member of the unit's management staff and one other individual, with both being familiar with the procedure.

13.5.1 Administrative Procedure Hierarchy

The procedure hierarchy contains four levels of administrative documents for use:

- A. Policy documents, which contain the highest level of direction for a Functional Area.
- B. Description documents which define processes and programs that provide for the control of nuclear operations
- C. Administrative and Technical Procedures which incorporate regulatory requirements and commitments. Administrative Procedures provide direction in areas that are common to all on-site organizations. Administrative procedures also address at a minimum, those administrative procedures specified in Regulatory Guide 1.33, Appendix A, item 1. Station-Specific Administrative Procedures (SA or HC) provide direction in areas that are unique to a specific station(s).
- D. Training and Reference Materials (T&RMs) provide direction for specific activities that are within a Functional Area's scope of responsibility or between Functional Areas with similar functional responsibilities or that control administrative functions between a limited number of departments. T&RMs comply with all applicable requirements specified in procedures.

Included in this group of procedures are operationally oriented administrative procedures which provide guidelines for the Shift Managers and their shift crews as well as procedures for "Daily Orders" usage and control. These procedures meet the requirements of 10CFR50.54(i), (j) (l) and (m).

Additional topics for administrative procedures may be addressed as required, and material may be shifted between specific procedures as needed. Topics for administrative procedures are listed in Table 13.5-1.

13.5.2 Operating and Maintenance Procedures

The operating and maintenance procedures meet the relevant requirements as discussed in Section 1.8 for Hope Creek and Appendix 3A for Salem of the respective UFSAR.

13.5.2.1 Main Control Room Operating Procedures

The following categories delineate those procedures that are performed primarily within the main control room. Operator familiarization with these procedures is acquired through initial, re-qualification and replacement training programs. Furthermore, these procedures will be utilized in simulator training.

13.5.2.1.1 System Operating Procedures

The procedures for startup, operation and shutdown of safety-related systems will be called System Operating Procedures (SOPs). SOPs will be developed to cover the operating activities listed in Regulatory Guide 1.33, Appendix A, item 4.

13.5.2.1.2 General Plant Operating Procedures

The general plant operating procedures will be called integrated operating procedures (IOPs) and will include the following procedures:

- Refueling to Cold Shutdown
- Preparation for Plant Startup
- Startup from Cold Shutdown to Rated Power
- Shutdown from Rated Power to Cold Shutdown
- Cold Shutdown to Refueling
- Power Changes During Operations
- Operations from Hot Standby

13.5.2.1.3 Procedures for Combating Emergencies and other Significant Events

The procedures for combating emergencies and other significant events will be

broken down into two categories. The Emergency Operating Procedures (EOPs) will be developed from the BWR and PWR Owners Group Emergency Procedure Guidelines, and the Abnormal Operating Procedures (AOPs) will cover the additional items in Regulatory Guide 1.33, Appendix A, item 6. The EOPs include the procedures developed from the BWR and PWR Owners Group Emergency Procedure Guidelines and comply with NUREG-0737, Supplement 1, Section 7.0.

Abnormal Operating Procedures are listed in Table 13.5-2.

13.5.2.1.4 Alarm Response Procedures

Alarm response procedures guide operators in their response to main control room alarm conditions. The alarm system consists of control room overhead annunciators, console pushbutton alarms, computer (digital) alarms and local/back panel alarms. A color code system (red, amber and white) is utilized for prioritizing control room overhead annunciators. The computer and local panel alarms are associated with an overhead annunciator. The priority of these alarms would be the same as the associated overhead alarm.

The alarm response procedures will be available in the main control room for the operators use. These procedures will be compiled in a manner which is consistent with the alarm system layout in the control room. For example, the overhead annunciator response procedures will be indexed by window box identification number.

13.5.3 Additional Operating and Maintenance Procedures

The following categories delineate those procedures that are performed primarily outside the limits of the main control room.

13.5.3.1 Chemistry Procedures

These procedures include chemical and radiochemical analysis, sample collection and equipment instruction. Chemistry procedures maintain coolant quality and concentrations of harmful agents within prescribed limits.

13.5.3.2 Emergency Plan Procedures

The Emergency Plan procedures define emergency response actions required to ensure public health and safety. These procedures include the actions necessary to mitigate radiological incidents, abnormal operational events such as fires, as well as natural hazards and civil occurrences.

13.5.3.3 Radiation Protection Procedures

Radiation protection procedures govern the implementation of the Radiation Protection Program described in Section 12.5. These procedures address access control and radiation work permits, contamination control, personnel monitoring, training and qualifications, radiological surveillance, respiratory protection, internal dose assessment, dose reduction(ALARA) and radioactive material control.

13.5.3.4 Instrument and Control Procedures

Instrument and control procedures govern the required periodic calibration and testing of plant instrumentation and other instrument maintenance. These procedures have provisions for taking the instrument out of service, ensuring accuracy adequate to maintain safety parameters, recording the date, as-found condition, corrective action(s), as-left condition, restoration of the instrument to normal operating status and identity of personnel performing the test.

13.5.3.5 Maintenance Procedures

Maintenance procedures provide guidelines for the maintenance of mechanical and electrical equipment in a satisfactory operational condition. This category also includes procedures for implementation of the preventive maintenance program for mechanical and electrical equipment. In addition, they provide for calibration and testing of protective relays. These procedures have provisions for recording the date, as-found condition, corrective action(s), as-left condition and identity of personnel performing the test.

13.5.3.6 Material Control Procedures

Material control procedures establish guidelines for the proper procurement, documentation, and control of materials and components associated with Q, F or R designated structures or systems. These procedures are sufficiently detailed to ensure that materials and components are purchased and handled in a controlled manner in accordance with 10CFR50, Appendix B.

13.5.3.7 Radwaste Management Procedures

Radwaste procedures include operating procedures for the solid, liquid and gaseous radwaste systems, and provide administrative controls for the shipment of solid radwaste and release of liquid and gaseous radwaste. These controls include adherence to 10CFR71, and provisions that allow for liquid and gaseous radwaste releases only when required data, analyses and approvals are completed.

13.5.3.8 Reactor Engineering Procedures

Reactor engineering procedures provide for the monitoring and evaluation of core thermal and hydraulic parameters. In addition, these procedures establish methods for evaluating fuel exposure, isotopic composition, core flux levels, and nuclear instrumentation setpoints as they relate to core power and flow.

13.5.3.9 Records Procedures

Records procedures provide for the identification, preparation and retention of plant records. These procedures also address storage requirements and retention periods.

13.5.3.10 Security Procedures

Security procedures describe and implement security requirements for the plants. These procedures include, but are not necessarily limited to, the subjects listed in Section 4.2 of ANSI N18.17-1973.

13.5.3.11 Surveillance Procedures

Surveillance procedures provide for operability verification of safety-related structures and components in accordance with the Technical Specifications. Surveillances are scheduled for performance in compliance with the frequencies established in the Technical Specifications.

13.5.3.12 Training Procedures

Training procedures establish guidelines to ensure that fully qualified personnel are responsible for the operation, maintenance and technical aspects of the plant. These procedures describe the training for licensed operators, licensed operator re-qualification, non-licensed personnel training and training for each discipline described in ANSI/ANS 3.1-1981.

13.5.3.13 Fire Protection Procedures

Fire protection procedures cover various aspects of fire safety, such as control of combustibles, control of ignition sources, periodic inspections of fire protection equipment, fire brigade training, fire drills and control of hazardous operations. These procedures also cover fire fighting organization, activities during a fire emergency and individual responsibilities during a fire emergency.

13.5.4 Technical Requirements Manual (TRM)

The Technical Requirements Manual (TRM) contains technical requirements and/or supporting information (e.g., tables and component lists) which were once contained in the SGS Technical Specifications (TS) (i.e., Appendix A of the SGS Facility Operating License). Removal of the TS and information is approved by the NRC through individual TS amendments. The TRM is intended to provide operational guidance and requirements for various plant conditions, actions, and testing similar to TS, however, these requirements are in accordance with licensing commitments. These changes add the TRM into the scope of procedures to be processed through the Station Qualified Reviewer (SQR) process and reviewed by PORC. Future changes to the relocated requirements and supporting information are processed in accordance with section 17.2 of the UFSAR, and are subject to a 10CFR50.59 Review. All non-editorial changes are reviewed by PORC.

The TRM is comprised of an index, the individual specification and bases. The manual is intended to provide a single location for the relocated TS items as a convenience for operations and other station personnel. The individual sections of the TRM contain the relocated licensing commitments which are subject to the provisions of 10CFR50.59 described above, and are controlled in accordance with the applicable established procedure process.

TABLE 13.5-1

ADMINISTRATIVE PROCEDURE TOPICS

- Administration Program Description
- Supplemental Workforce Program Description
- Business Planning Process Description
- Cost Management Process
- Project Evaluation and Authorization Process
- Budgeting Process Description
- Forecasting Process Description
- Salem/Peach Bottom Co-Owner Relationship
- Configuration Control Process Description
- Nonconforming Materials, Parts, or Components
- Outsourcing Engineering Work Authorization Process
- Margin Management
- Configuration Management
- Chemistry Process Description
- Environmental Program
- Environmental Management System Process
- Chemical Management Process
- Emergency Preparedness Process Description
- Equipment Reliability Process Description
- PSEG New Hire Engineer Mentoring Program
- Equipment Reliability Program Description
- Integrated Equipment Reliability Long Term Planning Process Description
- Material Degradation Management Program (MDMP)
- Regulatory Medical Testing Human Resources and Administration
- Exempt Overtime Human Resources and Administration

Note:

This list consists of <u>all</u> of the active "description documents" (which correspond to the prior Nuclear Administrative Procedures (NAPs) which were the top-level procedures from our prior procedure system).

It is a full-scale listing of the change from the old program to the new O.E.M. Program.

This list represents all of the documents in alpha-numeric order without using the document numbers themselves, since the list is by TOPICS.

TABLE 13-5-1 (Continued)

- <u>Protected Activities and Selection Processes Human Resources And Administration</u>
- Change Management Human Performance Policy
- Technical Human Performance Process Description
- Nuclear Cyber Security Program Description
- Nuclear Wireless Regulatory Compliance Program Description
- <u>Licensing Process Descriptions</u>
- Process Description Interface with Other Outside Agencies
- Reload Control Process Description
- Special Nuclear Material Control Process Description
- Quality Assurance Topical Report (QATR)
- Nuclear Oversight Audit Process Description
- Nuclear Oversight Performance Assessment Process Description
- Nuclear Oversight Vendor Audit (NOVA) Process Description
- Independent Inspection Process Description
- Independent Safety Engineering Function Process
- Conduct of Operations Process Description
- Project Management
- Contractor Utilization
- Contractor Alliance Process
- Records Management (RM) Process Description
- Radiation Protection Process Description
- External Dose Control Program Description
- Internal Dose Control Program Description
- Respiratory Protection Program Description
- Radioactive Material Control Program Description

TABLE 13-5-1 (Continued)

- High Radiation Area Program Description
- Radioactive Material/Waste Transportation/Disposal Program Description
- Radwaste Process Description
- Physical Protection Process Description
- Radioactive Contamination Control Program Description
- ALARA Program Description
- Radiological Instrumentation Program Description
- Radiological Posting and Labeling Program Description
- Fitness For Duty Process Description
- Access Authorization Process Description
- Training System Development Process Description
- Work Management Process Description

TABLE 13.5-2 ABNORMAL OPERATING PROCEDURES

- Loss of Protective System Channels
- Acts of Nature
- Fire Protection System Activation Plant Fires
- Fuel Cladding Failure
- Abnormal Release of Radioactivity
- Turbine Trip
- Generator Load Reject
- Plant Chemistry Out-of Specification
- Condenser Tube Leakage
- Dropped Control Rod
- Failed Open Safety/Pressure Relief Valve
- Control Room Evacuation
- Loss Of Electrical Power (Blackout)
- Loss of Primary Containment Integrity
- Loss of Secondary Containment Integrity (Hope Creek)
- Loss of Circulating Water
- Loss of Condenser Vacuum
- Loss of Recirculation Pump or Flow (Hope Creek)
- Loss of Generator Stator Water Cooling
- Loss of Generator Seal Oil
- Loss of Service Air

TABLE 13-5-2 (Continued)

- Loss of Instrument Air
- Loss of Shutdown Cooling
- Loss of Main Turbine Lube Oil
- Loss of Reactor Auxiliary Cooling Water (Hope Creek)
- Loss of Safety and Turbine Auxiliary Cooling Water (Hope Creek)
- Rod Control System Malfunction
- Loss of Feedwater or Feedwater Control
- Service Water System Malfunction
- Reactor Pressure Control System Malfunction
- Reactor Vessel Level Control System Malfunction (Hope Creek)
- Neutron Monitoring System Malfunction
- Fuel Pool Cooling
- Fuel Pool Cleanup System Malfunction (Hope Creek)
- Standby Liquid Control System Initiation (Hope Creek)

13.6 PLANT RECORDS

13.6.1 Plant History

The records of the preoperational phase of plant operation have been compiled and filed with the station records and are available for reference. These records include the results and analysis of preoperational testing, initial fuel loading, low power level tests and power escalation tests prior to commercial operation. Following the conduct of a test using a prepared test procedure, results are maintained in the station records.

13.6.2 Operating Records

Records concerning facility operations are maintained in the form of logs, log sheets, charts, and other such internal reports as are needed to document pertinent operating conditions, test and inspections.

The Operations Department logs constitute an important part of the operating history. The record in these logs is regarded as the authentic record of occurrences. Entries are made promptly, accurately and completely, and the logs are initialed or signed and dated by authorized personnel or otherwise authenticated.

An automatic Data Logger provides an hourly printout of various plant parameters. A computer is available for making calculations pertaining to plant performance, core power distribution and reactor power output.

13.6.3 Event Records

The plant instrument recorder charts are marked to indicate unusual or significant occurrences. The records of operation are reviewed daily and unusual occurrences are noted and corrections made as required. The minimum records to be kept are identified and retained in accordance with the Technical Specifications.

13.6.4 Maintenance and Testing Records

Each department maintains records to show, in detail, both preventive and

corrective maintenance to all major equipment assigned to their department.

13.6.5 Additional Records

In addition to the station operation, maintenance and test records, personal

radiation exposure records, environmental monitoring records and chemistry

records are maintained.

Records relating to radiation and radioactive materials are retained in

accordance with the requirements of 10CFR20 and the New Jersey Radiation

Protection Code.

Personnel radiation exposure records provide information on current exposure of

personnel in controlled areas, total accumulated exposures for all site

personnel, bioassay results for internal deposition of radioactive materials

and other measured or calculated data in regard to exposure of personnel to

radioactive materials. Radiation Protection surveys of radiation levels and

radioactive contamination shall be maintained on appropriate record forms.

Releases of wastes to the environment are monitored for radioactivity and

records of the results maintained.

Inspections and survey records of shipments and receipt of radioactive

materials as required by the Department of Transportation are kept on file.

13.6-2

SGS-UFSAR

Revision 19 November 19, 2001

13.7 SECURITY

The Security Plan describes a comprehensive program of physical protection for the facilities. Details of the Security Plan are considered Safeguards Information and are withheld from public disclosure. The plan addresses the following topics:

- 1. Physical security organization
- 2. Physical barriers
- 3. Access requirements
- 4. Detection aids
- 5. Communication requirements
- 6. Testing and maintenance
- 7. Response requirements.