

OPERATING DATA REPORT

DOCKET NO. 50-289
 DATE December 15, 1980
 COMPLETED BY D. G. Mitchell
 TELEPHONE (717) 948-8553

OPERATING STATUS

1. Unit Name: Three Mile Island Nuclear Station, Unit I
2. Reporting Period: November, 1980
3. Licensed Thermal Power (MWt): 2535
4. Nameplate Rating (Gross MWe): 871
5. Design Electrical Rating (Net MWe): 819
6. Maximum Dependable Capacity (Gross MWe): 840
7. Maximum Dependable Capacity (Net MWe): 776
8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons:

9. Power Level To Which Restricted, If Any (Net MWe):

10. Reasons For Restrictions, If Any:

	This Month	Yr.-to-Date	Cumulative
11. Hours In Reporting Period	720.	8040.	54769.
12. Number Of Hours Reactor Was Critical	0.0	0.0	31731.8
13. Reactor Reserve Shutdown Hours	0.0	0.0	839.5
14. Hours Generator On-Line	0.0	0.0	31180.9
15. Unit Reserve Shutdown Hours	0.0	0.0	0.0
16. Gross Thermal Energy Generated (MWF)	0.0	0.0	76531071.
17. Gross Electrical Energy Generated (MWH)	0.	0.	25484330.
18. Net Electrical Energy Generated (MWH)	0.	0.	23840053.
19. Unit Service Factor	0.0	0.0	56.9
20. Unit Availability Factor	0.0	0.0	56.9
21. Unit Capacity Factor (Using MDC Net)	0.0	0.0	55.3
22. Unit Capacity Factor (Using DER Net)	0.0	0.0	53.1
23. Unit Forced Outage Rate	100.0	100.0	34.3
24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):			

25. If Shut Down At End Of Report Period, Estimated Date of Startup:

26. Units In Test Status (Prior to Commercial Operation):

INITIAL CRITICALITY
 INITIAL ELECTRICITY
 COMMERCIAL OPERATION

Forecast

Achieved

AVERAGE DAILY UNIT POWER LEVEL

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 UNIT TMI-I
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MONTH November, 1980

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	0
2	0
3	0
4	0
5	0
6	0
7	0
8	0
9	0
10	0
11	0
12	0
13	0
14	0
15	0
16	0

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
17	0
18	0
19	0
20	0
21	0
22	0
23	0
24	0
25	0
26	0
27	0
28	0
29	0
30	0
31	0

UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH November

DOCKET NO. 50-289
 UNIT NAME TMI-I
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No.	Date	Type ¹	Duration (Hours)	Reason ²	Method of Shutting Down Reactor ³	Licensee Event Report #	System Code ⁴	Component Code ⁵	Cause & Corrective Action to Prevent Recurrence
1	11/1/80	F	720	D	1				Regulatory Restraint Order

¹
 F: Forced
 S: Scheduled

²
 Reason:
 A-Equipment Failure (Explain)
 B-Maintenance or Test
 C-Refueling
 D-Regulatory Restriction
 E-Operator Training & License Examination
 F-Administrative
 G-Operational Error (Explain)
 H-Other (Explain)

³
 Method:
 1-Manual
 2-Manual Scram.
 3-Automatic Scram.
 4-Other (Explain)

⁴
 Exhibit G - Instructions
 for Preparation of Data
 Entry Sheets for Licensee
 Event Report (LER) File (NUREG-
 0161)

⁵
 Exhibit I - Same Source

POOR ORIGINAL

OPERATING SUMMARY

The Unit was shutdown the entire month with core cooling provided by the Decay Heat Removal System.

At the beginning of the period the Reactor Coolant System was drained to a level of 10 to 15 inches above the centerline of the cold leg to perform maintenance work described below. This work was completed and the system was refilled on November 4, 1980.

During the week of November 17, 1980, the Secondary Plant was placed in service for the first time since shutdown following the Unit 2 accident. The purpose of this operation was primarily for Operations and Maintenance training and equipment checkout. The training value was extremely high and the effect on the morale of Unit 1 personnel was positive and probably the most worthwhile effect gained.

Each shift was able to perform startup and shutdown evolutions and observe the operation of all secondary systems and major components except the Moisture Separator Pumps and the Heater Drain Pumps. All equipment operated satisfactorily, however, necessary maintenance work was identified in areas such as powdex vessels, feedwater heater leakage, and circ water tunnel leakage.

A satisfactory vacuum reading of 27.5" hg was obtained and secondary system chemistry was within satisfactory operating levels within a reasonable period of time. Both Steam driven Main Feed Pumps were satisfactorily overspeed tested.

It was concluded that with exception of relatively minor maintenance work, the Secondary System is ready for restart.

MAJOR SAFETY RELATED MAINTENANCE

New internals were installed in High Pressure Injection (HPI) check valves MU-V-86A/B, 94, 95, 107A/B/C/D. The work involved was as follows.

1. Remove bonnet bolts and bonnet.
2. Take measurements and clean inside valve.
3. Fit machine new internals to valve.
4. Install new internals.
5. Final cleanup.
6. Reinstall seal ring and bonnet.
7. Torque bonnet studs/nuts.

OTSG "A" lower primary manway and lower primary handhole were removed for inspection of seating surface due to boron build-up on two of the studs/nuts. This work was performed with loop "A" of the RCS partially drained.

The following work was performed.

1. Clean room installed with particulate filter.
2. Insulation removed.

3. Lower manway studs/nuts removed.
4. Manway lowered in place.
5. Manway diaphragm removed.
6. Lower handhole studs/nuts removed.
7. Handhole removed.
8. Inspection/cleaning of seating surface.
9. Replacing of manway gasket and handhole gasket.
10. Installation of manway and handhole covers.
11. Torque of studs/nuts.

During this evolution, the manway studs were MT with satisfactory results.

During the week of November 17, 1980, the "Z" axis OTSG Auxiliary Feedwater nozzles were inspected as a result of cracking identified at other B&W facilities. No cracking was identified and additional inspections have been deemed unnecessary.

On November 8, 1980, the "A" Spent Fuel Cooling piping was drained to permit repair of a weld indication (#104) that was identified in the Intergranular Stress Corrosion Cracking investigation. At the end of November there remained six indications to be repaired before refilling the "A" SF piping.

REFUELING INFORMATION REQUEST

1. Name of Facility:

Three Mile Island Nuclear Station, Unit I

2. Scheduled date for next refueling shutdown:

Unknown

3. Scheduled date for restart following refueling:

Unknown

4. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment?

If answer is yes, in general, what will these be?

If answer is no, has the reload fuel design and core configuration been reviewed by your Plant Safety Review Committee to determine whether any unreviewed safety questions are associated with the core reload (Ref. 10 CFR Section 50.59)?

If no such review has taken place, when is it scheduled?

Amendment No. 50, Cycle 5 reload, was approved on 3-16-79.

5. Scheduled date (s) for submitting proposed licensing action and supporting information:

N/A

6. Important licensing considerations associated with refueling, e.g. new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures:

N/A

7. The number of fuel assemblies (a) in the core, and (b) in the spent fuel storage pool:

(a) 177

(b) 208

8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned, in number of fuel assemblies:

The present licensed capacity is 752. There are no planned increases at this time.

9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity:

1986 is the last refueling discharge which allows full core off-load capacity (177 fuel assemblies).