

UNITED STATES NUCLEAR REGULATOR COMMISSION WASHINGTON, D. C. 20555

PUBLIC SERVICE ELECTRIC AND GAS COMPANY PHILADELPHIA ELECTRIC COMPANY DELMARVA POWER AND LIGHT COMPANY ATLANTIC CITY ELECTRIC COMPANY

DOCKET NO. 50-311

SALEM NUCLEAR GENERATING STATION, UNITS NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 4 License No. DPR-75

- 1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Public Service Electric and Gas Company, Philadelphia Electric Company, Delmarva Power and Light Company and Atlantic City Electric Company (the licensees) dated November 20, 1980 complies with the standards and requirements of the Commission's regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
- Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C(2) of Facility Operating License No. DPR-75 is hereby amended to read as follows:

8012190

(2) <u>Lechnical Specifications</u>

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 4, are hereby incorporated in the license. Public Service Electric and Gas Company shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Avank Mirania

Frank J. Miragina, Acting Chief Licensing Branch No. 3 Division of Licensing

Attachment: Changes to the Technical Specifications

Date of Issuance: December 8, 1980

ATTACHMENT TO LICENSE AMENDMENT NC. 4

FACILITY LICENSE NO. DFR-75

DOCKET NO. 50-311

Revise Appendix A as follows:

Remove Pages	Insert Pages
5-2	6-2
6-3	6-3
6-6	6-6
6-7	0-7
6-8 4-0	6-9
6-11	6-11
6-12	6-12

Revise Appendix B as follows:

Remove Page	Insert Page
5-1	5-1



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6.1 RESPONSIBILITY

6.1.1 The Station Manager shall be responsible for overall facility operation and shall delegate in writing the succession to this responsibility during his absence.

5.2 ORGANIZATION

OFFSITE

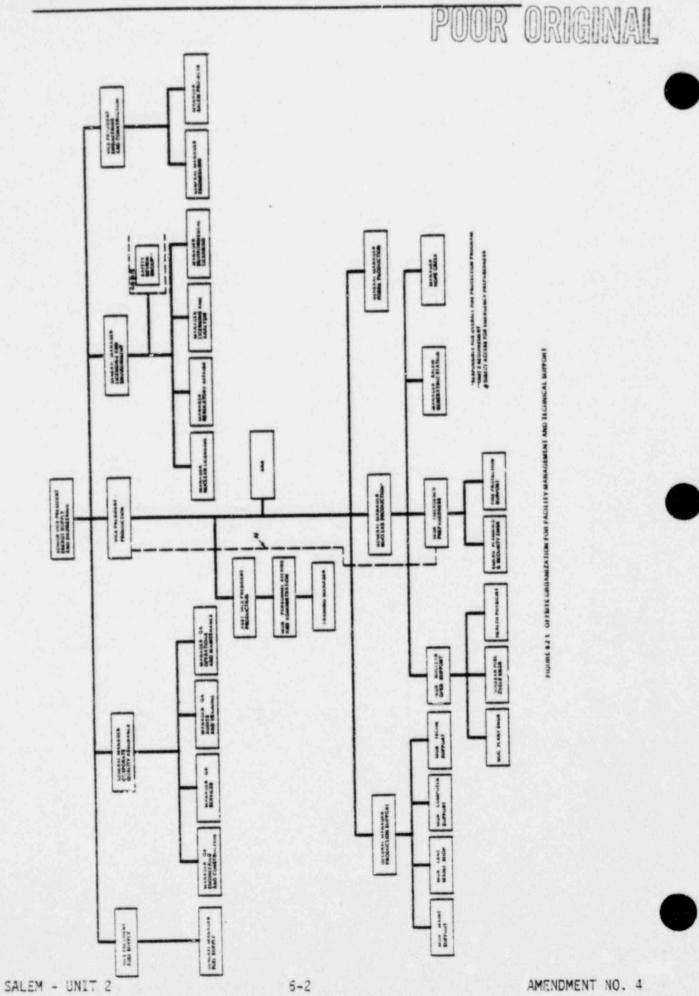
6.2.1 The offsite organization for facility management and technical support shall be as shown on Figure 6.2-1.

FACILITY STAFF

6.2.2 The Facility organization shall be as shown on Figure 6.2-2 and:

- a. Each on duty shift shall be composed of at least the minimum shift crew composition shown in Table 6.2-1.
- b. At least one licensed Operator shall be in the control room when fuel is in the reactor.
- c. At least two licensed Operators shall be present in the control room during reactor start-up, scheduled reactor shutdown and during recovery from reactor trips.
- d. A health physics technician shall be on site when fuel is in the reactor.
- e. All CORE ALTERATIONS shall be directly supervised by a licensed Senior Reactor Operator who has no other concurrent responsibilities during this operation.
- f. A site Fire Brigade of at least 5 members shall be maintained onsite at all times. The Fire Brigade shall not include 4 members of the minimum shift crew necessary for safe shutdown of the unit or any personnel required for other essential functions during a fire emergency.

[#]Fire Brigade composition may be 1 , than the minimum requirements for a period of time not to exceed 2 hours in order to accommodate unexpected absence of Fire Brigade members provided immediate action is taken to restore the Fire Brigade to within the minimum requirements.



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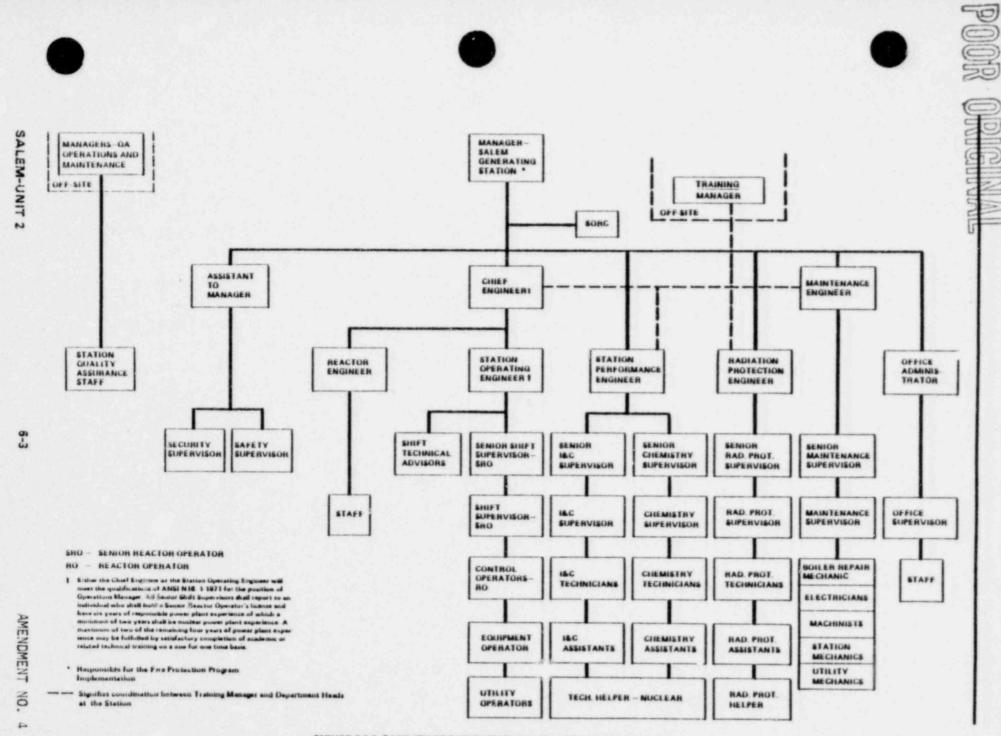


FIGURE 6.2.2 FACILITY ORGANIZATION-SALEM GENERATING STATION

TABLE 6.2-1

MINIMUM SHIFT CREW COMPOSITION

	APPLICABLE MODES	
LICENSE CATEGORY	1, 2, 3 & 4	5 & 6
SOL	2	1*
OL	2	1
Non-Licensed	2	1

Condition of Unit 2 - No Fuel in Unit 1

Condition of Unit 2 - Unit 1 in MODES 1, 2, 3 or 4

	APPLICABLE MODES	
LICENSE CATEGORY	1, 2, 3 & 4	5 & 6
SOL**	2	2*
OL	4	3***
Non-Licensed	4	3

Condition of Unit 2 - Unit 1 in MODES 5 or 6

	APPLICABLE MODES	
LICENSE CATEGORY	1, 2, 3 & 4	5 & 6
SOL**	2	1*
OL	3***	2
Non-Licensed	3	3

TABLE 6.2-1 (Continued)



*Does not include the licensed Senior Reactor Operator supervising CORE ALTERATIONS.

- **Each unit will be supervised by a shift supervisor who is a licensed SRO on that unit; or by a single individual if he is licensed on both units. The second senior operator licensed for each unit must be stationed in the control room area at all times when the unit is in operating modes 1 through 4; and could also be a single individual if he is licensed on Units 1 and 2.
- ***A reactor operator licensed for each unit must be at the controls of that unit at all times when fuel is in the reactor. Also, a relief reactor operator licensed for each unit must be available on-shift. This could be a single individual if he is licensed for both units.
 - #Shift crew composition may be less than the minimum requirements for a period of time not to exceed 2 hours in order to accomodate unexpected absence of on-duty shift crew members provided immediate action is taken to restore the shift crew composition to within the minimum requirements of Table 6.2-1.



5.3 FACILITY STAFF QUALIFICATIONS

6.3.1 Each member of the facility staff shall meet or exceed the minimum qualifications of ANSI N18.1-1971 for comparable positions. except for the Radiation Protection Engineer who shall meet or exceed the qualifications of Regulatory Guide 1.8, September 1975.

6.4 TRAINING

6.4.1 A retraining and replacement training program for the facility staff shall be coordinated by each functional level manager (Department Head) at the facility and maintained under the direction of the Training Manager and shall meet or exceed the requirements and recommendations of Section 5.5 of ANSI N18.1-1971 and Appendix "A" of 10 CFR Part 55.

5.5 REVIEW AND AUDIT

5.5.1 STATION OPERATIONS REVIEW COMMITTEE (SORC)

FUNCTION

6.5.1.1 The Station Operations Review Committee shall function to advise the Station Manager on all matters related to nuclear safety.

COMPOSITION

6.5.1.2 The Station Operations Review Committee shall be composed of the:

Chairman:	Chief Engineer
Vice Chairman:	Assistant to Manager
Vice Chairman:	Station Performance Engineer
Vice Chairman:	Maintenance Engineer
Member:	Station Operating Engineer
Member:	Reactor Engineer
Member:	Senior Shift Supervisor
Member:	Senior I & C Supervisor
Member:	Senior Chemistry Supervisor
Member:	Radiation Protection Engineer
Member:	Senior Maintenance Supervisor

ALTERNATES

6.5.1.3 All alternate members shall be appointed in writing by the SORC Chairman to serve on a temporary basis; however, no more than two alternates shall participate as voting members in SORC activities at any one time.

MEETING FREQUENCY

6.5.1.4 The SORC shall meet at least once per calendar month and as convened by the SORC Chairman or his designated alternate.

SALEM - UNIT 2



QUORUM

6.5.1.5 The minimum quorum of the SORC necessary for the performance of the SORC responsibility and authority provisions of these technical specifications shall consist of the Chairman or his designated alternate and four members including alternates.

RESPONSIBILITIES

6.5.1.6 The Station Operations Review Committee shall be responsible for:

- a. Review of 1) all procedures required by Specification 6.8 and changes thereto, 2) any other proposed procedures or changes thereto as determined by the Station Manager to affect nuclear safety.
- Review of all proposed tests and experiments that affect nuclear safety.
- c. Review of all proposed changes to Appendix "A" Technical Specifications.
- Review of all proposed changes or modifications to plant systems or equipment that affect nuclear safety.
- e. Investigation of all violations of the Technical Specifications including the preparation and forwarding of reports covering evaluation and recommendations to prevent recurrence to the General Manager - Nuclear Production and to the Chairman of the Nuclear Review Board.
- Review of events requiring 24 hour written notification to the Commission.
- .g. Review of facility operations to detect potential nuclear safety hazards.
- Performance of special reviews, investigations or analyses and reports thereon as requested by the Chairman of the Nuclear Review Board.
- Review of the Plant Security Plan and implementing procedures and shall submit recommended changes to the Chairman of the Nuclear Review Board.
- Review of the Emergency Plan and implementing procedures and shall submit recommended changes to the Chairman of the Nuclear Review Board.

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AUTHORITY

6.5.1.7 The Station Operations Review Committee shall:

- a. Recommend to the Station Manager written approval or disapproval of items considered under 6.5.1.6(a) through (d) above.
- b. Render determinations in writing with regard to whether or not each item considered under 6.5.1.6(a) through (e) above constitutes an unreviewed safety question.
- c. Provide written notification within 24 hours to the General Manager-Nuclear Production and the Nuclear Review Board of disagreement between the SORC and the Station Manager; however, the Station Manager shall have responsibility for resolution of such disagreements pursuant to 6.1.1 above.

RECORDS

6.5.1.8 The Station Operations Review Committee shall maintain written minutes of each meeting that, at a minimum, document the results of all SORC activities performed under the responsibility and authority provisions of these technical specifications. Copies shall be provided to the General Manager-Nuclear Production and Chairman of the Nuclear Review Board.

5.5.2 NUCLEAR REVIEW BOARD (NRB)

FUNCTION

6.5.2.1 The Nuclear Review Board shall function to provide independent review and audit of designated activities in the areas of:

- a. nuclear power plant operations
- nuclear engineering
- c. chemistry and radiochemistry
- d. metallurgy
- e. instrumentation and control
- f. radiological safety
- g. mechanical and electrical engineering
- h. quality assurance practices

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AMENCHENT NO. 4

COMPOSITION

6.5.2.2 The NRB shall be composed of the:

Chairman:	Assistant Vice President - Production
Vice Chairman:	Assistant to General Manager-Fuel Supply
Member:	General Manager-Fuel Supply
Member:	General Manager-Licensing and Environment
Member:	Manager-Nuclear Operations Support
	General Manager-Corporate QA
Member:	General Manager-Nuclear Production
Member:	Manager-Salem Generating Station
Member:	Principal Engineer
Member:	Managara Hara Creek
the second se	Manager-Hope Creek Generating Station

ALTERNATES

6.5.2.3 All alternate members shall be appointed in writing by the NRB Chairman to serve on a temporary basis; however, no more than two alternates shall participate as voting members in NRB activities at any one time.

CONSULTANTS

6.5.2.4 Consultants shall be utilized as determined by the NRB Chairman to provide expert advice to the NRB.



MEETING FREQUENCY

6.5.2.5 The NRB shall meet at least once per calendar quarter during the initial year of unit operation following fuel loading and at least once per six months thereafter.

QUORUM

6.5.2.6 The minimum quorum of NRB necessary for the performance of the NRB review and audit functions of these technical specifications shall consist of the Chairman or his designated alternate and at least 4 NRB members including alternates. No more than a minority of the quorum shall have line responsibility for operation of the facility.

REVIEW

6.5.2.7 The NRB shall review:

a. The safety evaluations for 1) changes to procedures, equipment or systems and 2) tests or experiments completed under the provision of Section 50.59, 10 CFR, to verify that such actions did not constitute an unreviewed safety question.

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- Proposed changes to procedures, equipment or systems which involve an unreviewed safety question as defined in Section 50.59, 10 CFR.
- c. Proposed tests or experiments which involve an unreviewed safety question as defined in Section 50.59, 10 CFR.
- Proposed changes to Technical Specifications or this operating license.
- Violations of codes, regulations, orders, Technical Specifications, license requirements, or of internal procedures or instructions having nuclear safety significance.
- Significant operating abnormalities or deviations from normal and expected performance of plant equipment that affect nuclear safety.
- g. Events requiring 24 hour written notification to the Commission.
- All recognized indications of an unanticipated deficiency in some aspect of design or operation of safety related structures, systems, or components.
- Reports and meetings minutes of the Station Operations Review Committee.

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AUDITS

5.5.2.8 Audits of facility activities shall be performed under the cognizance of the NRB. These audits shall encompass:

- a. The conformance of facility operation to provisions contained within the Technical Specifications and applicable license conditions at least once per 12 months.
- b. The performance, training and qualifications of the entire facility staff at least once per 12 months.
- c. The results of actions taken to correct deficiencies occurring in facility equipment, structures, systems or method of operation that affect nuclear safety at least once per 6 months.
- d. The performance of activities required by the Operational Quality Assurance Program to meet the criteria of Appendix "B", 10 CFR 50, at least once per 24 months.
- e. The Facility Emergency Plan and implementing procedures at least once per 24 months.
- The Facility Security Plan and implementing procedures at least once per 24 months.



- g. Any other area of facility operation considered appropriate by the NRB or the Vice President-Production.
- h. The Facility Fire Protection Program and implementing procedures at least once per 24 months.
- i. An independent fire protection and loss prevention program inspection and audit shall be performed at least once per 12 months utilizing either qualified offsite licensee personnel or an outside fire protection firm.
- j. An inspection and audit of the fire protection and loss prevention program shall be performed by a qualified outside fire consultant at least once per 36 months.

AUTHORITY

6.5.2.9 The NRB shall report to and advise the Vice President-Production on those areas of responsibility specified in Sections 6.5.2.7 and 6.5.2.8.

RECORDS

6.5.2.10 Records of NRB activities shall be prepared, approved and distributed as indicated below:

- a. Minutes of each NRB meeting shall be prepared, approved and forwarded to the Vice President-Production within 14 days following each meeting.
- b. Reports of reviews encompassed by Section 6.5.2.7 above, shall be prepared, approved and forwarded to the Vice President-Production within 14 days following completion of the review.
- c. Audit reports encompassed by Section 6.5.2.8 above, shall be forwarded to the Vice President-Production and to the management positions responsible for the areas audited within 30 days after completion of the audit.

6.6 REPORTABLE OCCURRENCE ACTION

6.6.1 The following actions shall be taken for REPORTABLE OCCURRENCES:

- a. The Commission shall be notified and/or a report submitted pursuant to the requirements of Specification 6.9.
- b. Each REPORTABLE OCCURRENCE requiring 24 hour notification to the Commission shall be reviewed by the SORC and submitted to the NRB and the General Manager-Nuclear Production.

SALEM - UNIT 2

6.7 SAFETY LIMIT VIOLATION

6.7.1 The following actions shall be taken in the event a Safety Limit is violated:

- a. The unit shall be placed in at least HOT STANDBY within one hour.
- b. The Safety Limit violation shall be reported to the Commission, the General Manager- Nuclear Production and to the NRB within 24 nours.
- c. A Safety Limit Violation Report shall be prepared. The report shall be reviewed by the SORC. This report shall describe (1) applicable circumstances preceding the violation, (2) effects of the violation upon facility components, systems or structures, and (3) corrective action taken to prevent recurrence.
- d. The Safety Limit Violation Report shall be submitted to the Commission, the NRB and the General Manager-Nuclear Production within 14 days of the violation.

6.8 PROCEDURES

6.8.1 Written procedures shall be established, implemented and maintained covering the activities referenced below:

- a. The applicable procedures recommended in Appendix "A" of Regulatory Guide 1.33, Revision 2, February 1978.
- Refueling operations.
- c. Surveillance and test activities of safety related equipment.
- d. Security Plan implementation.
- e. Emergency Plan implementation.
- f. Fire Protection Program implementation.

6.8.2 Each procedure and administrative policy of 6.8.1 above, and changes thereto, shall be reviewed by the SORC and approved by the Station Manager prior to implementation and reviewed periodically as set forth in administrative procedures.

5.8.3 Temporary changes to procedures of 6.8.1 above may be made provided:

- a. The intent of the original procedure is not altered.
- b. The change is approved by two members of the plant management staff, at least one of whom holds a Senior Reactor Operator's License on the unit affected.

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Administrative controls below apply to Appendix B - Part II only.

5 1 Responsibility

5.1.1 The implementation of the surveillance programs external to the plant, including sampling, sample analysis, evaluation of results and the preparation of required reports is the responsibility of the Licensing and Environment Department. This Department is responsible for the assignment of personnel to the above functions, for assurance that appropriate written procedures are utilized in the surveillance program activities and for assuring the quality of surveillance program results.

5.1.2 The Station Manager or his delegated alternate is responsible for operating the plant in compliance with the limiting conditions for operation as specified in the Environmental Technical Specifications and for the in-plant monitoring necessary to ensure such operation. His responsibility includes assurance that plant activities are conducted in such a manner as to provide continuing protection to the environment and that personnel performing such activities use appropriate written procedures as described in Section 5.5.

5.2 Review and Audit

5.2.1 Nuclear Review Board (NRB)

The NRB shall have the following responsibilities concerning the environmental impact of the plant:

- 1. NRB shall review:
 - Proposed Environmental Technical Specifications changes or license amendments.
 - b. Violations of Environmental Technical Specifications.
 - c. Environmental Monitoring Program and Evaluations.
 - d. Routine and non-routine reports required by the Environmental Technical Specifications.
- The NRB shall, at least once each year, conduct (or cause to have conducted) and evaluate audits of:
 - Plant operation to assure Environmental Technical Specification compliance.