(7-77)

LICENSEE EVENT REPORT

CONTROL BLOCK: (1) (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

0 1 | M A P P S 1 (2) | 0 0 - 0 0 0 0 - 0 0 (3) | 4 1 1 1 1 (4) | (5)

LICENSEE CODE | LICENSE NUMBER | LICENSE TYPE | CAT

CON'T

0 1 | REPORT SOURCE L (6) | 0 5 0 - 0 2 9 3 (7) | 1 1 1 3 8 0 (8) | 1 1 2 8 8 0 (9)

DOCKET NUMBER | EVENT DATE | REPORT DATE

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

0 2 | On November 13, 1980 during an inspection of equipment within the Drywell, operating

0 3 | personnel observed water leaking from a 2 inch reactor vessel drain line which connects

0 4 | with the RWCU system. Closer inspection revealed that the leaks origin was a 2 inch

0 5 | pipe coupling weld. The reactor which was critical at the time of the inspection

0 6 | was shutdown. The pipe was repaired prior to reactor being made critical on

0 7 | November 14, 1980.

0 8 |

0 9 | SYSTEM CODE C G (11) | CAUSE CODE B (12) | CAUSE SUBCODE C (13) | COMPONENT CODE P I P E X X (14) | COMP. SUBCODE A (15) | VALVE SUBCODE Z (16)

(17) LER/RO REPORT NUMBER: EVENT YEAR 8 0 | — | SEQUENTIAL REPORT NO. 0 8 4 | / | OCCURRENCE CODE 0 1 | REPORT TYPE X | — | REVISION NO. 1

ACTION TAKEN B (18) | FUTURE ACTION Z (19) | EFFECT ON PLANT C (20) | SHUTDOWN METHOD A (21) | HOURS 0 0 2 4 (22) | ATTACHMENT SUBMITTED (23) | NPRO-4 FORM SUB. N (24) | PRIME COMP. SUPPLIER N (25) | COMPONENT MANUFACTURER G 0 8 0 (26)

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

1 0 | The faulty weld was isolated, ground out and the coupling rewelded. A level three

1 1 | examiner, assisting in the repair, detected slag inclusions in the weld. The

1 2 | presence of this slag predestined failure of the weld. This failure is the result

1 3 | of a fabrication defect during construction.

1 4 |

1 5 | FACILITY STATUS C (28) | % POWER 0 1 0 (29) | OTHER STATUS (30) NA | METHOD OF DISCOVERY A (31) | DISCOVERY DESCRIPTION (32) Operational Event

1 6 | ACTIVITY RELEASED Z (33) | CONTENT OF RELEASE Z (34) | AMOUNT OF ACTIVITY (35) NA | LOCATION OF RELEASE (36) NA

1 7 | PERSONNEL EXPOSURES NUMBER 0 0 0 (37) | TYPE Z (38) | DESCRIPTION (39) NA

1 8 | PERSONNEL INJURIES NUMBER 0 0 0 (40) | DESCRIPTION (41) NA

1 9 | LOSS OF OR DAMAGE TO FACILITY TYPE Z (42) | DESCRIPTION (43) NA

2 0 | PUBLICITY ISSUED N (44) | DESCRIPTION (45) NA | NRC USE ONLY

8012100576

M. Thomas McLoughlin | 617-746-7900

NAME OF PREPARER | PHONE:

BOSTON EDISON COMPANY
PILGRIM NUCLEAR POWER STATION
DOCKET NO. 50 - 293
Attachment to LER 80-084/01X-1

On November 13, 1980, operating personnel observed leakage in the drywell. A closer inspection revealed that a socket weld coupling had failed. The reactor was shutdown until repairs could be performed.

Weld number B-4-77 RI on a 2" socket weld coupling connection to a 2" Schedule 80 pipe on the Reactor Water Cleanup System located at approximately the 40 foot level in the drywell failed. The 2" stainless steel pipe was repaired using a PNPS approved welding procedure. A liquid penetrant examination was performed prior to excavating the area of the failure. The excavation extended to within approximately .025 inches of the interior wall. A filler weld was then performed consuming the remaining backup material to ensure a sound weld and enable sufficient penetration. Prior to acceptance of the weld, a final visual and liquid penetrant examination and a 950 psi hydrostatic test were performed.

In addition to inspections and repairs of the failed weld a liquid penetrant examination was performed on the opposite end weld of the socket coupling. No degradation of this weld was detected.

A level three inspector observed slag inclusion in the weld material during excavation of weld B-4-77RI which is believed to be the primary cause of this event.

This failure was caused by a fabrication defect during construction and is not the result of thermal stress induced corrosion.