YANKEE ATOMIC ELECTRIC COMPANY

Regulatory Suppl File Cy.

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I STUART STREET, BOSTON, MASSACHUSETTS 02116



November 27, 1967

U. S. Atomic Energy Commission Washington, D. C. 20545

Attention: Division of Reactor Licensing

Dear Sirs:

We are enclosing herewith for your use and information twenty five copies of Operation Report No. 82 covering operations at the plant for the month of October, 1967.

Very truly yours,

Manager of Operations

WPJ:mj

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This report covers the operation of the Yankee Atomic Electric Company plant at Rowe, Massachusetts for the month of October, 1967.

At the beginning of the period the plant was operating at approximately 173 MWe. Thereafter plant load varied between 169 MWe and 177 MWe as circulating water inlet temperature varied between 48° F and 61° F.

On October 17, slowly increasing steam generator chloride concentrations again indicated the presence of a main condenser tube leak. Preparations were made for a plant load reduction and condenser tube plugging to be accomplished at approximately 2100 hours October 19. However, at 2015 hours October 19 the Z-126 115 KV line was reported to have a glowing disconnect at the Harriman Station. A load reduction to 450 MWt was initiated immediately and various safety procedures consistent with the single 115 KV line plant operation were implemented. Shortly thereafter the load reduction was continued to 90 MWe in order to conduct the condenser tube leak test and plugging. As a result of the test four leaking tubes in the vicinity of the air intake area were found and plugged, fifteen additional tubes in the same area were also plugged as a precautionary measure. Subsequently, during the plant shutdown on October 28, the condenser was re-entered and three previously plugged tubes were re-opened for axial determination of the leak location. This was accomplished by permitting a stringcaptive stopper to be drawn into the tube by the condenser vacuum. In all cases the leak was found to be within three to five feet from the circulating water inlet tube sheet. By 2245 hours October 19 the condenser had been closed out and plant load increased to 450 MWt, the load still limited by difficulties on the Z-126 line which had been isolated at 2140 hours. At 0220 hours the Z-126 line was reported clear and plant load increase to 600 MWt was initiated immediately thereafter.

On October 4 a vapor container inspection revealed that the low set pressurizer safety valve had developed minor leakage past the seat. This was detected by feeling the near down stream piping. Leakage cannot be detected at the piping external to the vapor container. At the end of the reporting period no increase in leak rate could be detected either by an increase in safety valve discharge piping temperature or pressurizer heater power demand.

Cleaning, inspection, and decontamination of the spent fuel pit continued throughout the reporting period. Subsequent to the removal of both aluminum fuel storage racks, the wooden fuel rack platforms were removed and cut up for disposal. These platforms had apparently suffered considerable radiation damage. Although the wood appeared to be sound it was embrittled and readily susceptible to fracture under minimal transverse loadings. At the end of the period new cypress platforms were