UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

PHILADELPHIA ELECTRIC COMPANY
PUBLIC SERVICE ELECTRIC AND GAS COMPANY
DELMARVA POWER AND LIGHT COMPANY
ATLANTIC CITY ELECTRIC COMPANY

DOCKET NO. 50-277

PEACH BOTTOM ATOMIC POWER STATION, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 73
License No. DPR-44

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The applications for amendment by Philadelphia Electric Company, et al. (the licensee) dated May 23 and July 16, 1980, comply with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the applications, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C.(2) of Facility Operating License No. DPR-44 is hereby amended to read as follows:

   (2) Technical Specifications

   The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 73, are hereby incorporated in the license. PECO shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Robert W. Reid, Chief
Operating Reactors Branch #4
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: September 22, 1980

ATTACHMENT TO LICENSE AMENDMENT NO. 73

FACILITY OPERATING LICENSE NO. DPR-44

DOCKET NO. 50-277

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by amendment number and contain vertical lines indicating the area of change.

Remove Page	Insert Page
180	180
186	186
187	187
234f	234f
234n	234n
-	234o (new page)
257a	257a

TABLE 3.7.1 (Cont'd.)

PRIMARY CONTAINMENT ISOLATION VALVES

Group	Valve Identification	Number of Power Operated Valves		Maximum Operating Time (sec.)	Normal Position	Action on Initiating Signal
		Inboard	Outboard			
NA	Feedwater check valves	2*	2*	NA	O	Process
3	Radioactive gas sample isolation valves	4	1#	NA	C	SC
3	Instrument nitrogen compressor suction line isolation valves	1#	1	5	O	GC
3	Oxygen Analyzer System		14	NA	O	GC
NA	Standby liquid control system check valves	1*	1*	NA	C	Process
2B	RHRS shutdown cooling suction isolation valves	1	1	32	C	SC
2B	RHRS shutdown cooling injection isolation valves		1	24	C	SC
2B	RHRS Reactor Vessel Head Spray isolation valves	1	1	30	C	SC
2C	Feedwater Flush Valves		2	50	C	SC
4	HPCIS steam line isolation valves	1	1	20	O	GC
5	RCICS steam line isolation valves	1	1	15	O	GC
2A	Reactor water cleanup system isolation valves	1	1	30	O	GC
2A	Reactor water cleanup system return isolation valve		1	20	O	GC

*valves not power operated.
#Effective upon completion of the modification required by NUREG 0578, Item 2.1.5a

Amendment No. 60, 61, 73

TABLE 3.7.4 (Cont'd.)

PRIMARY CONTAINMENT TESTABLE ISOLATION VALVES

Pen No.		Notes
22	AO-2969A; Check Valve	(1)(2)(4)(5)(10)
25	AO-2520; AO-2505; AO-2519; AO-2521A; AO-2521B	(1)(2)(4)(5)(9)
25	AO-2523; Check Valves	(1)(2)(4)(5)
26	AO-2506; AO-2507	(1)(2)(4)(5)(9)
26	SV-2671G; SV-2978G	(1)(2)(4)(5)
26	AO-2509; AO-2510; AO-4235	(1)(2)(4)(5)(9)
26	SV-4960B; SV-4961B; SV-4966B	(1)(2)(4)(5)
26	SV-8100*	(1)(2)(4)(5)
26, 51C 203, 219	SV-8101*	(1)(2)(4)(5)
39A	MO-10-31B; MO-10-26B	(1)(2)(4)(5)(9)
39A	SV-4949B; SV-4948B	(1)(2)(4)(5)
39B	MO-10-31A; MO-10-26A	(1)(2)(4)(5)(9)
39B	SV-4949A; SV-4948A	(1)(2)(4)(5)
41	AO-2-39; AO-2-40	(1)(2)(4)(5)(9)
42	Check Valve 11-16, XV-14A, XV-14B	(1)(2)(4)(5)(10)
51A	SV-2671E; SV-2978E	(1)(2)(4)(5)
51B	SV-2671D; SV-2978D	*
51C	SV-2671C; SV-2978C	*
51C	SV-4960C; SV-4961C; SV-4966C	*
51D	SV-2980; Check Valve	*
52F	AO-2969B; Check Valve	(1)(2)(4)(5)(10)
57	AO-2-316; AO-2-317	(1)(2)(4)(5)(9)
102B	Breathing Air System-2 Gate Valves*	(1)(2)(4)(5)(9)
203	SV-2671B; SV-2978B	(1)(2)(4)(5)

TABLE 3.7.4 (Cont'd.)

PRIMARY CONTAINMENT TESTABLE ISOLATION VALVES

Pen No.		Notes
203	SV-4960D; SV-4961D; SV-4966D	(1) (2) (4) (5)
203A	AO-2502B; Check Valve 9-26B	(1)(2)(4)(5)(9)
205B	AO-2502A; Check Valve 9-26A	*
211A	MO-10-38B; MO-10-39B; MO-10-34B	(1)(2)(4)(5)(9)
211A	SV-4951B; SV-4950B	(1)(2)(4)(5)
211B	MO-10-38A; MO-10-39A; MO-10-34A	(1)(2)(4)(5)(9)
211B	SV-4951A; SV-4950A	(1)(2)(4)(5)
212	Check Valve 13-50; AO-4240; AO-4241	(1)(2)(4)(5)(9)
214	Check Valve 23-65; AO-4247; AO-4248	(1)(2)(4)(5)(9)
217B	MO-4244; MO-4244A	(1)(2)(4)(5)(9)
218A	AO-2968	(1)(2)(4)(5)(10)
218B	SV-2671A; SV-2978A	(1)(2)(4)(5)
219	AO-2511; AO-2512; AO-2513; AO-2514	(1)(2)(4)(5)(9)
219	SV-2671F; SV-2978F; SV-4960A SV-4961A; SV-4966A	(1)(2)(4)(5)
221	Check Valve 13-38	(1)(2)(4)(5)(9)
223	Check Valve 23-56	(1)(2)(4)(5)(9)
225	MO-13-41; MO-13-39	(1)(2)(4)(5)(9)
225	MO-14-70; MO-14-71	(1)(2)(4)(5)(9)
227	MO-23-58; MO-23-57	(1)(2)(4)(5)(9)

* Effective upon completion of the modification

TABLE 3.11.D.1

Safety Related Shock Suppressors (Snubbers)

SNUBBER NUMBER	LOCATION	ELEVATION	SNUBBER IN HIGH (1) RADIATION AREA DURING SHUTDOWN	SNUBBERS ESPECIALLY DIFFICULT TO REMOVE	SNUBBERS INACCESSIBLE DURING NORMAL OPERATION	SNUBBERS ACCESSIBLE DURING NORMAL OPERATION
				Unit 2 (cont'd)		
1-GG-S-35	MSRV	155	See 4.11.D.4.b	X	Drywell	
1-GG-S-36	MSRV	155	"	X	Drywell	
1-GG-S-63	MSRV	155	"	X	Drywell	
1-GG-S-64	MSRV	155	"	X	Drywell	
1-GG-S-65	MSRV	155	"	X	Drywell	
1-GG-S-66	MSRV	155	"	X	Drywell	
1-GG-S-67	MSRV	155	"	X	Drywell	
1-GG-S-68	MSRV	155	"	X	Drywell	
1-GG-S-69	MSRV	155	"	X	Drywell	
1-GG-S-72	MSRV	155	"	X	Drywell	
1-GG-S-74	MSRV	155	"	X	Drywell	
1-GG-S-75	MSRV	155	"	X	Drywell	
1-GG-S-76	MSRV	155	"	X	Drywell	
1-GG-S-77	MSRV	155	"	X	Drywell	

TABLE 3.11.D.1

Safety Related Shock Suppressors (Snubbers)

SNUBBER NUMBER	LOCATION	ELEVATION	SNUBBER IN HIGH (1) RADIATION AREA DURING SHUTDOWN	SNUBBERS ESPECIALLY DIFFICULT TO REMOVE	SNUBBERS INACCESSIBLE DURING NORMAL OPERATION	SNUBBERS ACCESSIBLE DURING NORMAL OPERATION
23-DBN-S-28	HPCI	117	See 4.11.D.4.b	X		Torus Room
23-DBN-S-29	HPCI	117	"	X		Torus Room
23-HB-S-30	HPCI	93	"			HPCI Room
23-HB-S-36	HPCI	103	"			HPCI Room
23-HB-S-37	HPCI	103	"			HPCI Room
23-HB-S-38	HPCI	126	"	X		Torus Room
1-GG-S-101-A	MSRV	135	"	X	Drywell	
1-GG-S-101-B	MSRV	135	"	X	Drywell	
1-GG-S-102-A	MSRV	135	"	X	Drywell	
1-GG-S-102-B	MSRV	135	"	X	Drywell	
1-GG-S-103-A	MSRV	135	"	X	Drywell	
1-GG-S-103-B	MSRV	135	"	X	Drywell	
1-GG-S-104-A	MSRV	135	"	X	Drywell	
1-GG-S-104-B	MSRV	135	"	X	Drywell	
1-GG-S-105-A	MSRV	135	"	X	Drywell	
1-GG-S-105-B	MSRV	135	"	X	Drywell	
1-GG-S-106-B	MSRV	135	"	X	Drywell	

Amendment No. 35, 55, 73

TABLE 3.11.D.1

Safety Related Shock Suppressors (Snubbers)

SNUBBER NUMBER	LOCATION	ELEVATION	SNUBBER IN HIGH (1) RADIATION AREA DURING SHUTDOWN	SNUBBERS ESPECIALLY DIFFICULT TO REMOVE	SNUBBERS INACCESSIBLE DURING NORMAL OPERATION	SNUBBERS ACCESSIBLE DURING NORMAL OPERATION
1-GG-S-106-B	MSRV	135	See 4.11.D.4.b	X	Drywell	
1-GG-S-108-A	MSRV	145	"	X	Drywell	
1-GG-S-108-B	MSRV	145	"	X	Drywell	
1-GG-S-109-A	MSRV	145	"	X	Drywell	
1-GG-S-109-B	MSRV	145	"	X	Drywell	
1-GG-S-110-A	MSRV	145	"	X	Drywell	
1-GG-S-110-B	MSRV	145	"	X	Drywell	
1-GG-S-111-A	MSRV	145	"	X	Drywell	
1-GG-S-111-B	MSRV	145	"	X	Drywell	
1-GG-S-112-A	MSRV	145	"	X	Drywell	
1-GG-S-112-B	MSRV	145	"	X	Drywell	

Notes for Table 3.11.D.1

1) Modifications to this table due to changes in high radiation areas should be submitted to the NRC as part of the next license amendment.

6.9.3 Continued

e. Release rate of Radioactive Effluents, Specification 3.8.B.7, 3.8.C.3.b, 3.8.C.5.

f. Sealed source leakage in excess of limits, Specification 3.13.2

g. Effluent Releases

Effluent data should be summarized monthly, except in instances when more data is needed, and the items listed below reported semi-annually on the standard form "Report of Radioactive Effluents".

1) Gaseous Releases

a) Total radioactivity released (in curies) of noble and activation gases.

b) Maximum noble gas release rate during any one-hour period.

UNITED STATES
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PHILADELPHIA ELECTRIC COMPANY
PUBLIC SERVICE ELECTRIC AND GAS COMPANY
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ATLANTIC CITY ELECTRIC COMPANY

DOCKET NO. 50-278

PEACH BOTTOM ATOMIC POWER STATION, UNIT NO. 3

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 72
License No. DPR-56

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The applications for amendment by Philadelphia Electric Company, et al. (the licensee) dated May 23 and July 16, 1980, comply with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the applications, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C.(2) of Facility Operating License No. DPR-56 is hereby amended to read as follows:

   (2) Technical Specifications

   The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 72, are hereby incorporated in the license. PECO shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Robert W. Reid, Chief
Operating Reactors Branch #4
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: September 22, 1980

ATTACHMENT TO LICENSE AMENDMENT NO. 72

FACILITY OPERATING LICENSE NO. DPR-56

DOCKET NO. 50-278

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by amendment number and contain vertical lines indicating the area of change.

Remove Page	Insert Page
180	180
186	186
187	187
257a	257a

TABLE 3.7.1 (Cont'd.)

PRIMARY CONTAINMENT ISOLATION VALVES

Group	Valve Identification	Number of Power Operated Valves		Maximum Operating Time (sec.)	Normal Position	Action on Initiating Signal
		Inboard	Outboard			
NA	Feedwater check valves	2*	2*	NA	O	Process
3	Radioactive gas sample isolation valves	4	1#	NA	C	SC
3	Instrument nitrogen compressor suction line isolation valves	1#	1	5	O	GC
3	Oxygen Analyzer System		14	NA	O	GC
NA	Standby liquid control system check valves	1*	1*	NA	C	Process
2B	RHRS shutdown cooling suction isolation valves	1	1	32	C	SC
2B	RHRS shutdown cooling injection isolation valves		1	24	C	SC
2B	RHRS Reactor Vessel Head Spray isolation valves	1	1	30	C	SC
2C	Feedwater Flush Valves		2	50	C	SC
4	HPCIS steam line isolation valves	1	1	20	O	GC
5	RCICS steam line isolation valves	1	1	15	O	GC
2A	Reactor water cleanup system isolation valves	1	1	30	O	GC
2A	Reactor water cleanup system return isolation valve		1	20	O	GC

*valves not power operated.
#Effective upon completion of the modification required by NUREG 0578, item 2.1.5a

Amendment No. 60, 61, 72

TABLE 3.7.4 (Cont'd.)

PRIMARY CONTAINMENT TESTABLE ISOLATION VALVES

Pen No.		Notes
22	AO-3969A; Check Valve	(1)(2)(4)(5)(10)
25	AO-3520; AO-3505; AO-3519; AO-3521A; AO-3521B	(1)(2)(4)(5)(9)
25	AO-3523; Check Valves	(1)(2)(4)(5)
26	AO-3506; AO-3507	(1)(2)(4)(5)(9)
26	SV-3671G; SV-3978G	(1)(2)(4)(5)
26	AO-3509; AO-3510; AO-5235	(1)(2)(4)(5)(9)
26	SV-5960B; SV-5961B; SV-5966B	(1)(2)(4)(5)
26	SV-9100*	*
26, 51C 203, 219	SV-9101*	*
39A	MO-10-31B; MO-10-26B	(1)(2)(4)(5)(9)
39A	SV-5949B; SV-5948B	(1)(2)(4)(5)
39B	MO-10-31A; MO-10-26A	(1)(2)(4)(5)(9)
39B	SV-5959A; SV-5948A	(1)(2)(4)(5)
41	AO-2-39; AO-2-40	(1)(2)(4)(5)(9)
42	Check Valve 11-16, XV-14A, XV-14B	(1)(2)(4)(5)(10)
51A	SV-3671E; SV-3978E	(1)(2)(4)(5)
51B	SV-3671D; SV-3978D	*
51C	SV-3671C; SV-3978C	*
51C	SV-5960C; SV-5961C; SV-5966C	*
51D	SV-3980; Check Valve	*
52F	AO-3969B; Check Valve	(1)(2)(4)(5)(10)
57	AO-2-316; AO-2-317	(1)(2)(4)(5)(9)
102B	Breathing Air System-2 Gate Valves*	*
203	SV-3671B; SV-3978B	(1)(2)(4)(5)

TABLE 3.7.4 (Cont'd.)

PRIMARY CONTAINMENT TESTABLE ISOLATION VALVES

Pen No.		Notes
203	SV-5960D; SV-5961D; SV-5966D	(1) (2) (4) (5)
205A	AO-3502B; Check Valve 9-26B	(1)(2)(4)(5)(9)
205B	AO-3502A; Check Valve 9-26A	*
211A	MO-10-38B; MO-10-39B; MO-10-34B	(1)(2)(4)(5)(9)
211A	SV-5951B; SV-5950B	(1)(2)(4)(5)
211B	MO-10-38A; MO-10-39A; MO-10-34A	(1)(2)(4)(5)(9)
211B	SV-5951A; SV-5950A	(1)(2)(4)(5)
212	Check Valve 13-50; AO-5240; AO-5241	(1)(2)(4)(5)(9)
214	Check Valve 23-65; AO-5247; AO-5248	(1)(2)(4)(5)(9)
217B	MO-5244; MO-5244A	(1)(2)(4)(5)(9)
218A	AO-3968	(1)(2)(4)(5)(10)
218B	SV-3671A; SV-3978A	(1)(2)(4)(5)
219	AO-3511; AO-3512; AO-3513; AO-3514	(1)(2)(4)(5)(9)
219	SV-3671F; SV-3978F; SV-5960A SV-5961A; SV-5966A	(1)(2)(4)(5)
221	Check Valve 13-38	(1)(2)(4)(5)(9)
223	Check Valve 23-56	(1)(2)(4)(5)(9)
225	MO-13-41; MO-13-39	(1)(2)(4)(5)(9)
225	MO-14-70; MO-14-71	(1)(2)(4)(5)(9)
227	MO-23-58; MO-23-57	(1)(2)(4)(5)(9)

* Effective upon completion of the modification.

6.9.3 Continued

e. Release rate of Radioactive Effluents, Specification 3.8.B.7, 3.8.C.3.b, 3.8.C.5.

f. Sealed source leakage in excess of limits, Specification 3.13.2

g. Effluent Releases

Effluent data should be summarized monthly, except in instances when more data is needed, and the items listed below reported semi-annually on the standard form "Report of Radioactive Effluents".

1) Gaseous Releases

a) Total radioactivity released (in curies) of noble and activation gases.

b) Maximum noble gas release rate during any one-hour period.