AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO.	50-313		
UNIT	1		
DATE	6-15-81		
COMPLETED BY	L.S. Bramlett		
TEL EPHONE	(501)968-2519		

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	825	17	824
2	826	18	824
3	827	19	824
4	825	20	822
5	825	21	821
6		22	820
7	827	23	821
8	827	24	822
9	828	25	
10	829	26	822
11	829	27	822
12	826	28	824
13	465	29	825
14	533	30	825
15	819	31	824
16	822	51	

INSTRUCTIONS

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On this format, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megaviatt.

THIS DOCUMENT CONTAINS POOR QUALITY PAGES (9/77)

OPERATING DATA REPORT

DOCKET NO. 50-313 DATE 5-15-81 COMPLETED BY L.S.Bramlett (5011968-2519 TELEPHONE

OPERATING STATUS

Unit Name: Arkansas Nuclear One - Un	nit 1 Notes
Reporting Period: May 1-31, 1981	
Licensed Thermal Power (MWT): 902.	74
Design Electrical Rating (Net MWe):850	
Maximum Dependable Capacity (Gross MWe):	883 836
. Maximum Dependable Capacity (Net MWe):	
. If Changes Occur in Capacity Ratings (Items Number 3 1	Through 7) Since Last Report, Give Reasons:

None

None

10. Reasons For Restrictions. If Any: ____

	This Month	Yrto-Date	Cumulative
11. Hours In Reporting Period	744.0	3623.0	56538.0
12. Number Of Hours Reactor Was Critical	734.7	1862.4	37652.7
13. Reactor Reserve Shutdown Hours	0.0	149.0	5044.0
14. Hours Generator On-Line	730.5	1786.0	36831.0
15. Unit Reserve Shutdown Hours	0.0	0.0	817.5
16. Gross Thermal Energy Generated (MWH)	1855601	4334598	88506564
17. Gross Electrical Energy Generated (MWH)	624288	1451530	29150091
18. Net Electrical Energy Generated (MWH)	597488	1386160	27801799
19. Unit Service Factor	98.2	49.3	65.1
20. Unit Availability Factor	98.2	49.3	66.6
21. Unit Capacity Factor (Using MDC Net)	96.1	45.8	58.8
22. Unit Capacity Factor (Using DER Net)	94.5	45.0	57.9
23. Unit Forced Outage Rate	1.8	2.0	16.7

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):

25. If Shut Down At End Of Report Period, Estimated Date of Startup:		
26. Units In Test Status (Prior to Commercial Operation):	Forecast	Achieved
INITIAL CRITICALITY		1 <u>1 1 1 1 1 1 1</u> 1
INITIAL ELECTRICITY	1999 <u>- 1999</u> - 1999	1997 <u>- 1997</u> (* 19
COMMERCIAL OPERATION		and the second second second

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REFUELING INFORMATION

DATE: June, 1981

1.	Name of facility. Arkansas Nuclear One - Unit 1
2.	Scheduled date for next refueling shutdown. 1/1/83
3.	Scheduled date for restart following refueling. 3/15/83
4.	Will refueling or resumption of operation thereafter require a technical specification change or other license amendment? If answer is yes, what, in general, will these be? If answer is no, has the reload fuel design and core configuration been reviewed by your Plant Safety Review Committee to determine whether any unreviewed safety questions are associated with the core reload (Ref. 10 CFR Section 50.59)?
	Yes. Reload report and associated pr posed Technical Specification
	changes. Also, the safety analysis of eight demonstration high
	burn-up assemblies will be provided.
5.	Scheduled date(s) for submitting proposed licensing action and supporting information. 10/1/82
6.	Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures.
	Will reload 72 fresh fuel assemblies and operate for approximately
	16 months. Eight of which will be high burn-up test assemblies.
7.	The number of fuel assemblies (a) in the core and (b) in the spent fuel storage pool. a) 177 b) 244
8.	The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned, in number of fuel assemblies.
	present 589 increase size by 0
	The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity.
	DATE:1986

UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH May 1981

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DOCKET NO.	50-313
UNIT NAME	ANO-Unit I
DATE	6-1-81 L.S.Bramlett
MPLETED BY TELEPHONE	(501)968-2519

No.	Date	Type ¹	Duration (Hours)	Reason ²	Method of Shutting Down Reactor 3	Licensee Event Report #	System Civde ⁴	Component Code ⁵	Cause & Corrective Action to Prevent Recurrence
81-05	810513	F	13.5	A	3	N/A	HH	Instru	A false condenser hotwell level signal caused the condensate pumps to trip, this caused the MFW pumps to trip on low suction pressure and in turn tripped the reactor on MFW pump anticipatory trip circuitry. The false signal will be investi- gated during a future outage to confirm preliminary results of a possible ground fault.
F: Fo S: Sct	rced neduled	B-Ma C-Re D-Re E-Op F-Ad G-Op	uipment Fa intenance o fueling gulatory Re	or Test estriction ning & U e rror (Ex	n License Exan	aination	3-Auto 4-Con	ual ual Scram. omatic Scram. tinuation ad Reducti	0161)

UNIT SHUTDO' NS AND POWER REDUCTIONS

INSTRUCTIONS

This report should describe all plant shutdowns during the report period. In addition, it should be the source of explanation of significant dips in average power levels. Each significant reduction in power level (greater than 20% reduction in average daily pow r level for the preceding 24 hours) should be noted, even though the unit may not have been shut down completely¹. For such reductions in power level, the duration should be listed as zero, the method of reduction should be listed as 4 (Other), and the Cause and Corrective Action to Prevent Recurrence column should be used to provide any needed explanation to fully describe the circumstances of the outage or power reduction.

NUMBER. This column should indicate the sequential number assigned to each shutdown or significant reduction in power for that calendar year. When a shutdown or significant power reduction begins in one report period and ends in another, an entry should be made for both eport periods to be sure all shutdowns or significant power reductions are reported. Until a unit has achieved its first power generation, no number should be assigned to each entry.

DATE. This column should indicate the date of the start of each shutdown or significant power reduction. Report as year, month, and day. August 14, 1977 would be reported as 770814. When a shutdown or significant power reduction begins in one report period and ends in another, an enery should be made for both report periods to be sure all shutdowns or significant power reductions are reported.

TYPE. Use "F" or "S" to indicate either "Forced" or "Scheduled," respectively, for each shutdown or significant power reduction. Forced shutdowns include those required to be initiated by no later than the weekend following discovery of an off-normal cor 4 tion. It is recognized that some judgment is required in categorizing shutdowns in this way. In general, a forced shutdown is one that would not have been completed in the absence of the condition for which corrective action was taken.

DURATION. Self-explanatory. When a shutdown extends beyond the end of a report period, count only the time to the end of the report period and pick up the ensuing down time in the following report periods. Report duration of outages rounded to the nearest tenth of an hour to facilitate summation. The sum of the total outage hours plus the hours the generator was on line should equal the gross hours in the reporting period.

REASON. Categorize by letter designation in accordance with the table appearing on the report form. If category H must be used, supply brief comments.

METHOD OF SHUTTING DOWN THE REACTOR OR REDUCING POWER. Categorize by number designation

¹Note that this differs from the Edison Electric Institute (EEI) definitions of "Forced Partial Outage" and "Scheduled Partial Outage." For these terms, EEI uses a change of 30 MW as the break point. For larger power reactors, 30 MW is too small a change to warrant explanation. in accordance with the table appearing on the report form. If category 4 must be used, supply brief comments.

LICENSEE EVENT REPORT \neq . Reference the applicable reportable occurrence pertaining to the outage or power reduction. Enter the first four parts (event year, sequential report number, occurrence code and report type) of the five part designation as described in Item 17 of Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161). This information may not be immediately evident for all such shutdowns, of course, since further investigation may be required to ascertain whether or not a reportable occurrence was involved.) If the outage or power reduction will not result in a reportable occurrence, the positive indication of this lack of correlation should be noted as not applicable (N/A).

SYSTEM CODE. The system in which the outage or power reduction originated should be noted by the two digit code of Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (N UREG-0161).

Systems that do not fit any existing code should be designated XX. The code ZZ should be used for those events where a system is not applicable.

COMPONENT CODE. Select the most appropriate component from Exhibit I - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161), using the following critieria:

- A. If a component failed, use the component directly involved
- B. If not a component failure, use the related component, e.g., wrong valve operated through error: list valve as component.
- C. If a chain of failures occurs, the first component to mulfunction should be listed. The sequence of events, including the other components which fail, should be described under the Cause and Corrective Action to Prevent Recurrence column.

Components that do not fit any existing code should be designated XXXXXX. The code ZZZZZZ should be used for events where a component designation is not applicable.

CAUSE & CORRECTIVE ACTION TO PREVENT PECUR-RENCE. Use the column in a narrative fashion to amplify or explain the circumstances of the shutdown or power reduction. The column should include the specific cause for each shut down or significant power reduction and the immediate and contemplated long term corrective action taken. if appropriate. This column should also be used for a description of the major safety-related corrective maintenance performed during the outage or power reduction including an identification of the critical path activity and a report of any single release of radioactivity or single radiation exposure specifically associated with the outage which accounts for more than 10 percent of the allowable annual values.

For long textual reports continue narrative on separate paper and reference the shutdown or power reduction for this narrative.

NRC MONTHLY OPERATING REPORT OPERATING SUMMARY - MAY 1981 UNIT I

The unit operated at 100% full power and continued there until 5/13/81. On this date the reactor tripped on the anticipatory trip circuitry for loss of feedwater pumps. The feedwater pumps tripped on low suction pressure when the condensate pumps tripped on a false condenser hotwell low level signal. The unit returned to 100% power operation on 5/15/81 and continued operation there through the remainder of the month.

A correction was made of 1.7 hours additional to the unit forced outage hours for the yearly totals and cumulative totals due to an error in the March 1981 report.

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