

ATTACHMENT I

PROPOSED TECHNICAL SPECIFICATION

CHANGE

RELATED TO

ORGANIZATON AND

MANAGEMENT TITLE CHANGES

POWER AUTHORITY OF THE STATE OF NEW YORK
JAMES A. FITZPATRICK NUCLEAR POWER PLANT
DOCKET NO. 50-333

POOR ORIGINAL

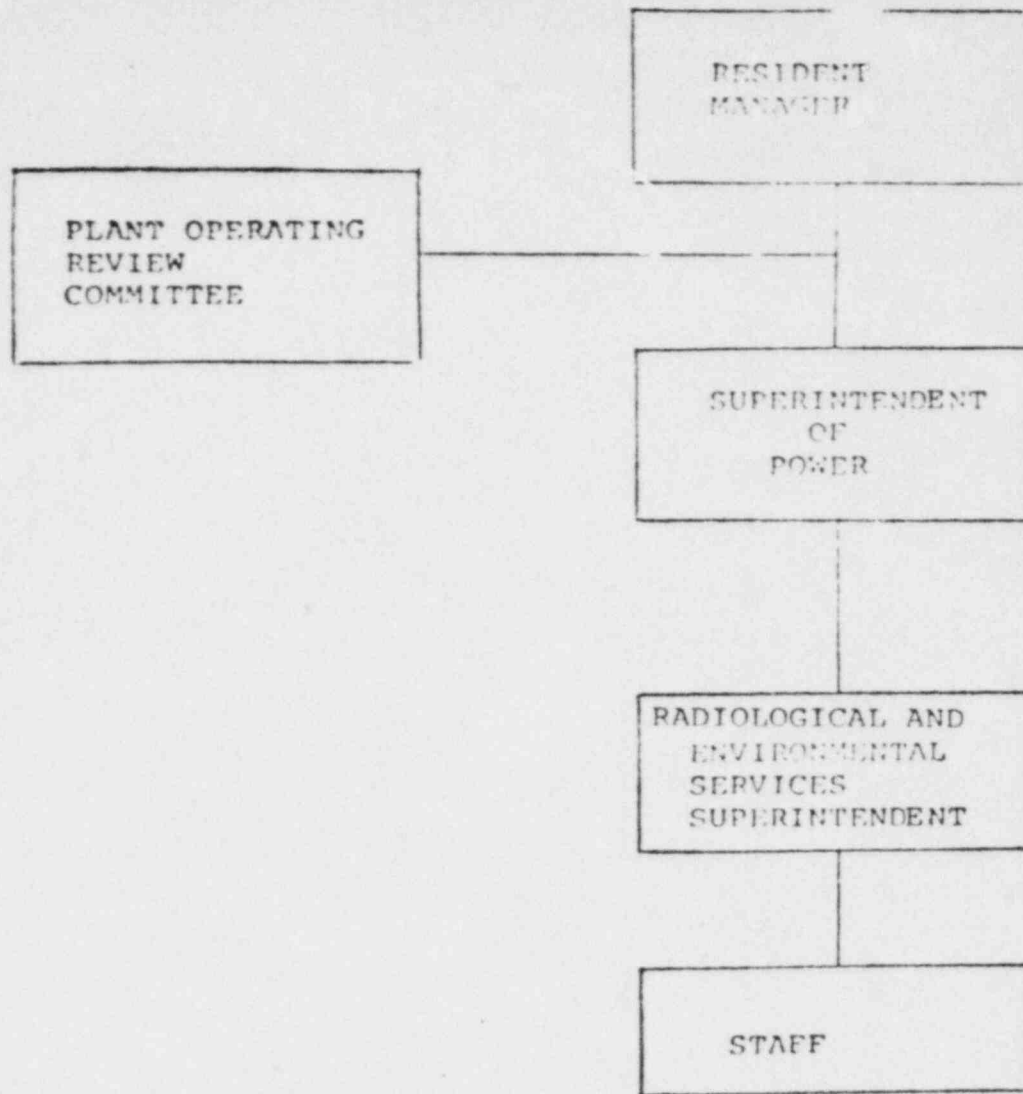


FIGURE 5.2-1
 POWER AUTHORITY OF THE STATE OF NEW YORK
 JAMES A. FITZPATRICK NUCLEAR POWER PLANT
 PLANT ORGANIZATION - RADIOLOGICAL ENVIRONMENTAL

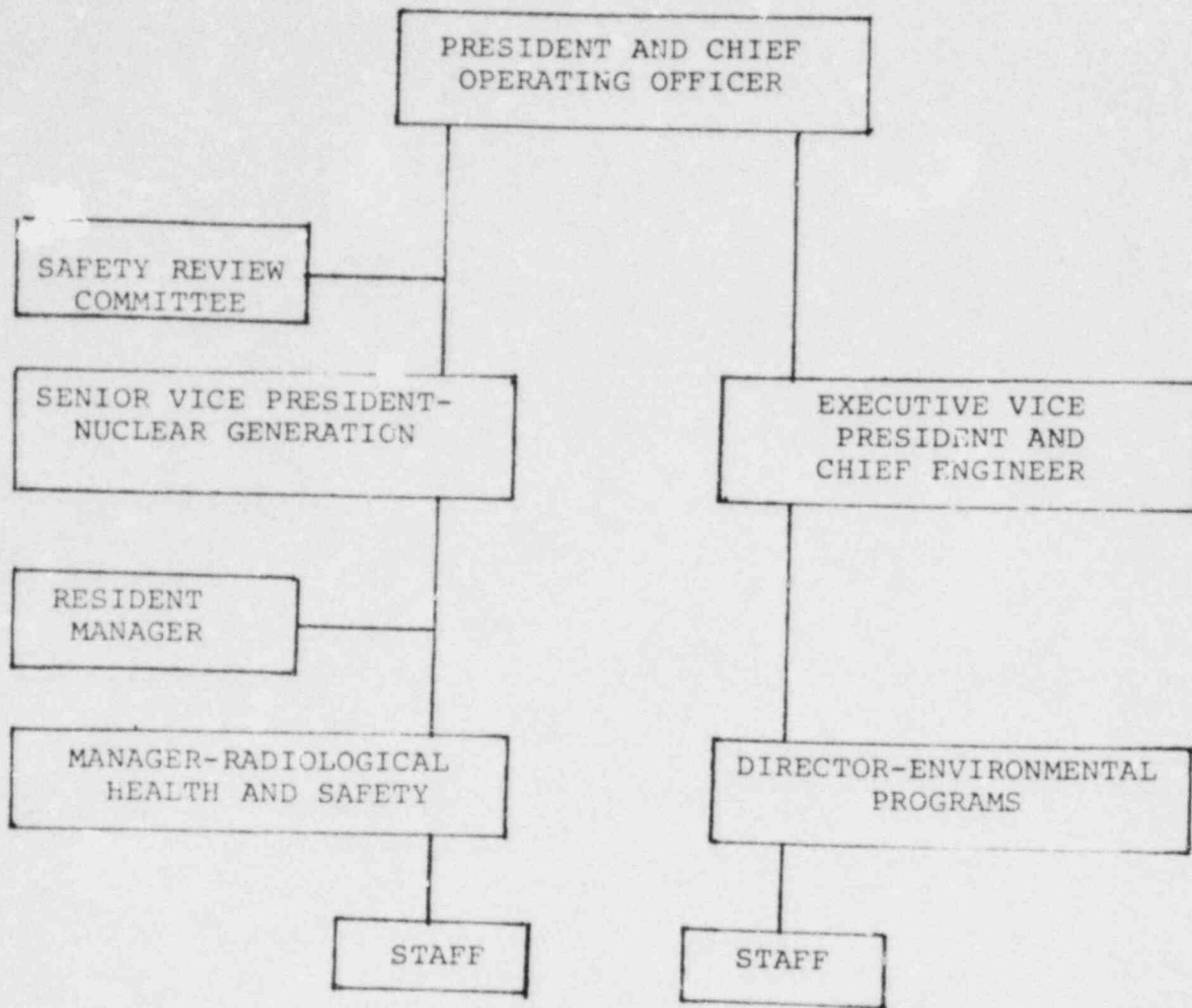


Figure 5.2-2
Management Organization-Offsite
Radiological Environmental
James A. FitzPatrick Nuclear
Power Plant

Administrative Controls are the means by which plant operations are subject to management control. Measures specified in this section provide for the assignment of responsibilities, plant organization, staffing qualifications and related requirements, review and audit mechanisms, procedural controls and reporting requirements. Each of these measures are necessary to ensure safe and efficient facility operation.

6.1RESPONSIBILITY

The Resident Manager is responsible for safe operation of the plant. During periods when the Resident Manager is unavailable, the Superintendent of Power will assume his responsibilities. In the event both are unavailable, the Resident Manager may delegate this responsibility to other qualified supervisory personnel. The Resident Manager reports directly to the Sr. Vice President Nuclear Generation, as shown in Fig. 6.1-1.

6.2PLANT STAFF ORGANIZATION

The plant staff organization is shown graphically in Fig. 6.2-1 and functions as follows:

1. A licensed senior reactor operator shall be on site at all times when there is fuel in the reactor.
2. In addition to item 1 above, a licensed reactor operator shall be in the control room at all times when there is fuel in the reactor.
3. In addition to items 1 & 2 above, a licensed reactor operator shall be readily available on site whenever the reactor is in other than cold condition.
4. Two licensed reactor operators shall be in the control room during start-ups and scheduled shutdowns.
5. A licensed senior reactor operator shall be responsible for all movement of new and irradiated fuel within the site boundary. A licensed reactor operator will be required to manipulate or directly supervise the manipulation of the controls of all fuel moving equipment, except the reactor building crane. All fuel movements by the reactor building crane, except new fuel movements from receipt through dry storage, shall be under the direct supervision of a licensed reactor operator. All fuel movements within the core shall be directly monitored by a member of the reactor analyst group. (a)

- f. Radiological safety
- Mechanical engineering
- h. Electrical engineering
- i. Administrative controls and quality assurance practices
- j. Environment
- k. Civil/Structural Engineering
- l. Nuclear Licensing
- m. Emergency Planning
- n. Other appropriate fields associated with the unique characteristics of a nuclear power plant

MEMBERSHIP

6.5.2.2 The SRC shall be composed of the following voting members:

Chairman:	Director-Nuclear Design and Analysis
Vice-Chairman:	Vice President-Quality Assurance
Member:	Manager-Nuclear Technical Support
Member:	Manager-Radiological Health and Safety
Member:	Senior Vice President-Nuclear Generation
Member:	Director-Electrical Design and Analysis
Member:	Director of Environmental Programs
Member:	Director-Civil/Structural Design and Analysis
Member:	Director-Mechanical Design and Analysis
Member:	Senior Inservice Inspection Engineer-Nuclear Technical Support
Member:	Manager-Nuclear Licensing
Member:	Senior Nuclear Engineer-Design and Analysis
Member:	Manager-Nuclear Operations

ALTERNATES

6.5.2.3 All alternate members shall be appointed in writing by the SRC Chairman; however, no more than two alternates shall participate as voting members in SRC activities at any one time.

CONSULTANTS

6.5.2.4 Consultants shall be utilized as determined by the SRC Chairman to provide expert advice to the SRC.

MEETING FREQUENCY

6.5.2.5 The SRC shall meet at least once per calendar quarter during the initial year of facility operation following initial fuel loading and at least once per six months, thereafter.

QUORUM

6.5.2.6 A quorum of SRC shall consist of the Chairman or Vice-Chairman and six members, including alternates. No more than minority of the quorum shall have a direct line responsibility for the operation of the plant.

REVIEW

6.5.2.7 The SRC shall review:

- a. The safety evaluation for 1) changes to procedures, equipment or systems and 2) tests or experiments completed under the provision of Section 50.59, 10 CFR, to verify that such actions did not constitute an unreviewed safety question.
- b. Proposed changes to procedures, equipment or systems which involve an unreviewed safety question as defined in Section 50.59, 10 CFR.
- c. Proposed tests or experiments which involve an unreviewed safety question as defined in Section 50.59, 10 CFR.
- d. Proposed changes to Technical Specifications of this Operating License.
- e. Violations of code, regulations, orders, Technical Specifications, license requirements, or of internal procedures or instructions having nuclear safety significance.
- f. Significant operating abnormalities or deviations from normal and expected performance of plant equipment that affect nuclear safety.
- g. Events requiring 24 hour written notification to the Commission.
- h. All recognized indications of an unanticipated deficiency in some aspect of design or operation of safety related structures, systems, or components.
- i. Reports and meetings minutes of the Plant Operating Review Committee.

AUDIT

6.5.2.8 Audits of facility activities shall be performed under the cognizance of the SRC. These audits shall encompass:

- a. The conformance of facility operation to provisions contained within the Technical Specifications and applicable license conditions at least once per 12 months.
- b. The performance, training and qualifications of the entire facility staff at least once per 12 months.

- c. The results of actions taken to correct deficiencies occurring in facility equipment, structures, systems or method of operation that affect nuclear safety at least once per 6 months.
- d. The performance of activities required by the Operational Quality Assurance Program to meet the criteria of Appendix "B", 10 CFR 50, at least once per 24 months.
- e. The Facility Emergency Plan and implementing procedures at least once per 24 months.
- f. The Facility Security Plan and implementing procedures at least once per 24 months.
- g. Any other area of facility operation considered appropriate by the SRC or the President.
- h. The Facility Fire Protection Program and implementing procedures at least once per two years.
- i. An independent fire protection and loss of prevention inspection and audit shall be performed annually utilizing either qualified offsite licensee personnel or an outside fire protection firm.
- j. An inspection and audit of the fire protection and loss prevention program shall be performed by an outside qualified fire consultant at intervals no greater than 3 years.

AUTHORITY

6.5.2.9 The SRC shall report to and advise the President and Chief Operating Officer (to be referred to in the Tech Spec. as the President) on those areas of responsibility specified in Section 6.5.2.7 and 6.5.2.8.

RECORDS

6.5.2.10 Records will be maintained in accordance with ANSI 18.7-1972. The following shall be prepared, approved and distributed as indicated below:

- a. Minutes of each SRC meeting shall be prepared, approved and forwarded to the President within 14 days after the date of the meeting.
- b. Reports of reviews encompassed by Section 6.5.2.7 above, shall be prepared, approved and forwarded to the President within 14 days following completion of the review.
- c. Audit reports encompassed by Section 6.5.2.8 above, shall be forwarded to the President and to the management positions responsible for the areas audited within 30 days after completion of the audit.

CHAPTER

6.5.2.11 Conduct of the committee will be in accordance with a charter approved by the President setting forth the mechanism for implementation of the committee's responsibilities and authority.

6.6 REPORTABLE OCCURRENCE ACTION

(A) In the event of a Reportable Occurrence, the NRC shall be notified and/or a report submitted pursuant to the requirements of Specification 6.9.

(B) Each Reportable Occurrence requiring 24 hours notification to the NRC shall be reviewed timely by the PORC and a report submitted by the Resident Manager to the Senior Vice President-Nuclear Generation and the SRC.

6.7 SAFETY LIMIT VIOLATION

(A) If a safety limit is exceeded, the reactor shall be shut down and reactor operation shall only be resumed in accordance with the provisions of 10 CFR 50.36 (c) (i).

(B) An immediate report of each safety limit violation shall be made to the NRC by the Resident Manager. The Senior Vice President-Nuclear Generation and Chairman of the SRC will be notified within 24 hours.

(C) The PORC shall prepare a complete investigative report of each safety limit violation and include appropriate analysis and evaluation of: (1) applicable circumstances preceding the occurrence, (2) effects of the occurrence upon facility components or structures and (3) corrective action required to prevent recurrence. The Resident Manager shall forward this report to the Senior Vice President-Nuclear Generation, Chairman of the SRC and the NRC.

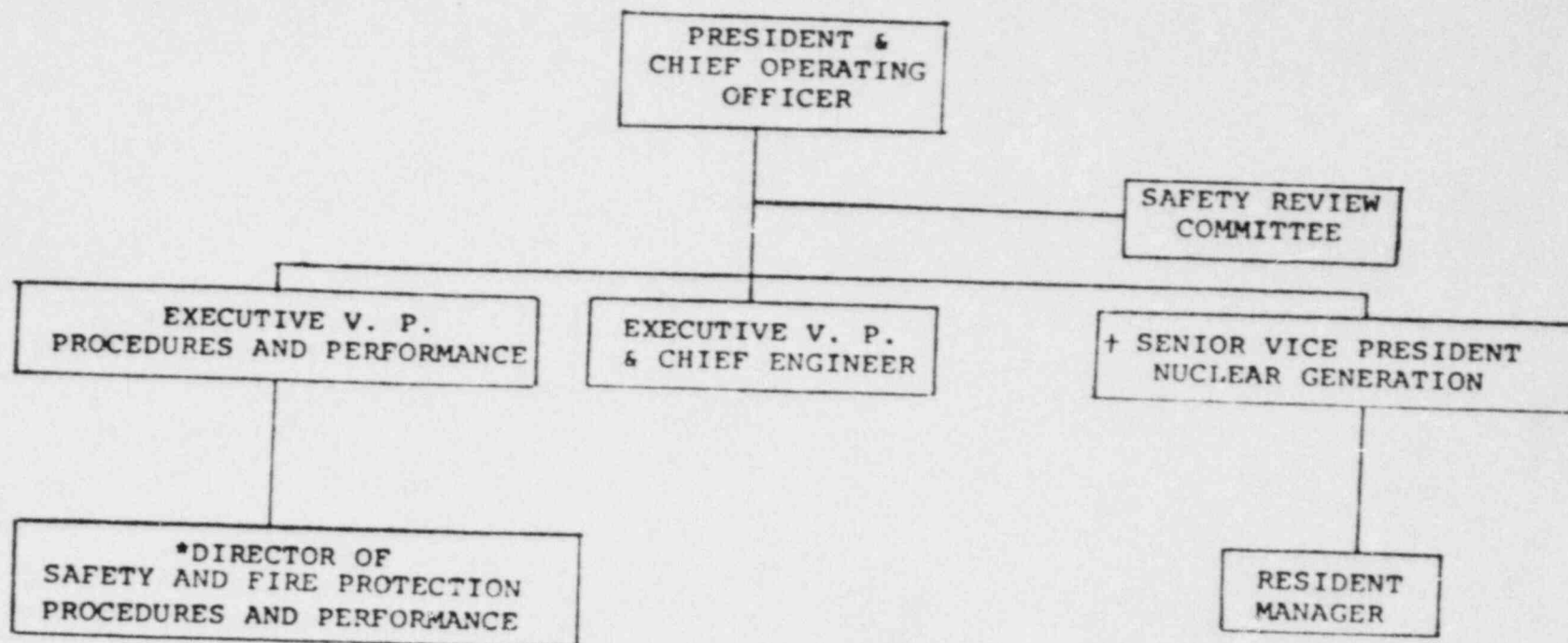
6.8 PROCEDURES

(A) Written procedures and administrative policies shall be established, implemented and maintained that meet or exceed the requirements and recommendations of Section 5 "Facility Administrative Policies and Procedures" of ANSI 18.7-1972 and Appendix A of Regulatory Guide 1.33, November 1972. In addition, procedures shall be established, implemented and maintained for the Fire Protection Program.

(B) Those procedures affecting nuclear safety shall be reviewed by PORC and approved by the Resident Manager prior to implementation.

(C) Temporary changes to nuclear related procedures may be made provided:

1. The intent of the original procedure is not altered.

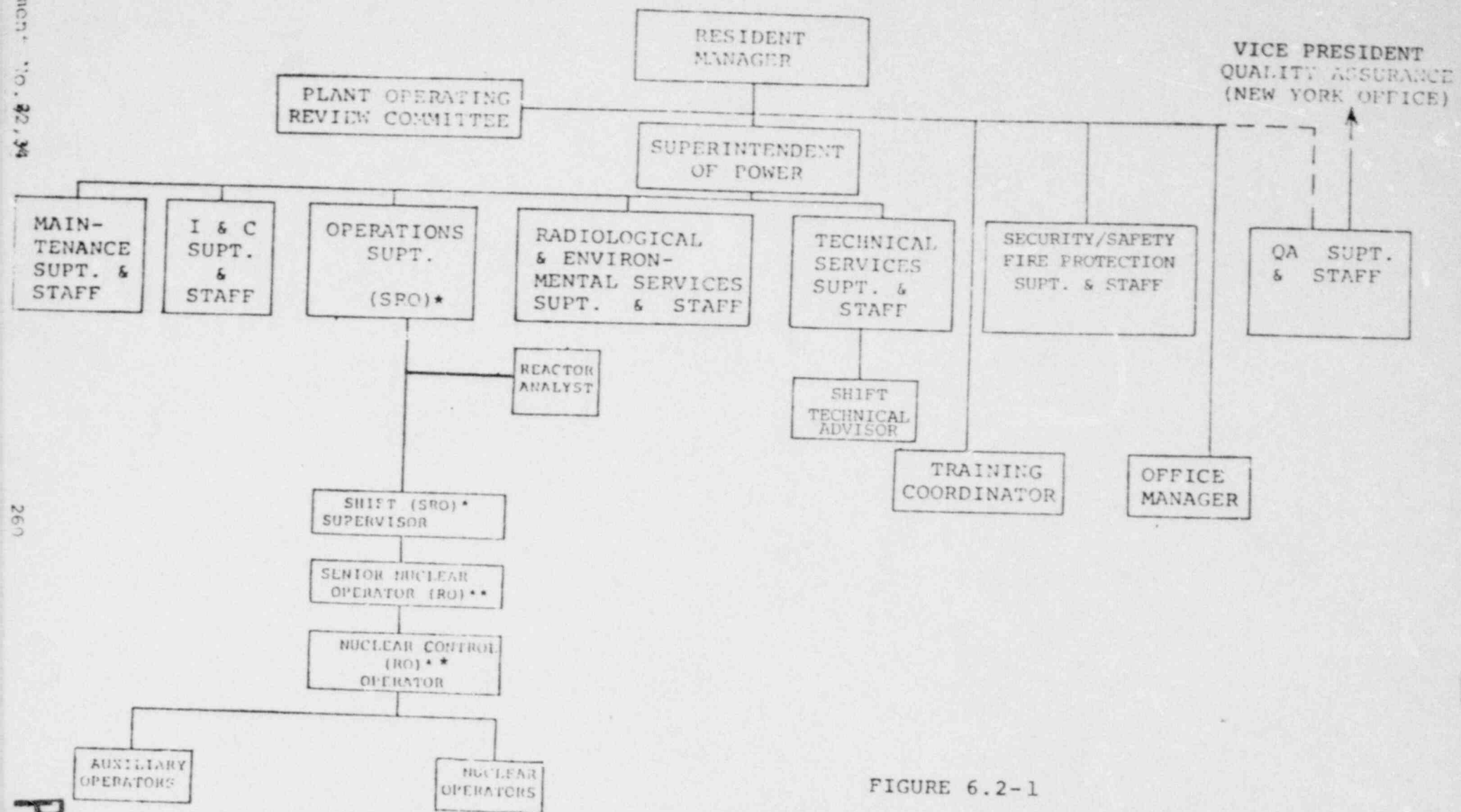


†The Senior Vice President Nuclear Generation is responsible for administering the fire protection program.

*The Director of Safety and Fire Protection is the off-site management position responsible for assessing the effectiveness of the fire protection programs by conducting reviews and audits.

FIGURE 6.1-1

MANAGEMENT ORGANIZATION CHART
JAMES A. FITZPATRICK NUCLEAR POWER PLANT



*SRO - SENIOR REACTOR OPERATOR
 **RO - REACTOR OPERATOR

FIGURE 6.2-1

POWER AUTHORITY OF THE STATE OF NEW YORK
 JAMES A. FITZPATRICK NUCLEAR POWER PLANT
 PLANT STAFF ORGANIZATION

ATTACHMENT II

SAFETY EVALUATION

RELATED TO

ORGANIZATION AND

MANAGEMENT TITLE CHANGES

POWER AUTHORITY OF THE STATE OF NEW YORK
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Section I - Description of Modification

The modification seeks to revise the Section 6.0 Administrative Controls of Appendix A by:

- 1) changing the titles of several members of the Safety Review Committee (SRC),
- 2) adding the Manager-Nuclear Operations to the membership of the SRC and increasing the quorum requirements,
- 3) adding Emergency Planning as a new area of SRC competence,
- 4) deleting the position of Executive Vice President and Director of Power Operations from the Management Organizational Chart (Figure 6.1-1),
- 5) changing the titles of several positions shown on the Management Organizational Chart (Figure 6.1-1),
- 6) changing the reporting requirements of the Senior Vice President Nuclear Generation and the Safety Review Committee to the President and Chief Operating Officer,
- 7) adding the position of Executive Vice President-Procedures and Performance to the Management Organizational Chart (Figure 6.1-1) and the Director of Safety and Fire Protection who is responsible for the Power Authority direction of the Fire Protection Program,
- 8) changing the title of several positions shown on the Plant Staff Organizational Chart (Figure 6.2-1), correction of a typographical error for the Nuclear Control Operators position and addition of the Shift Technical Advisor.

The modification seeks to revise Section 5.0 Administrative Controls of Appendix B by:

- 1) correcting typographical errors on Figure 5.2-1 Plant Organization to reflect the fact that this chart is for the radiological environmental program only,
- 2) to correct organizational responsibilities and reporting requirements and title changes in Figure 5.2-2 as described in Items 4 and 6 above,
- 3) to reflect the fact that this chart is for the radiological environmental program only by the deletion the Director-Environmental Programs and the addition of the Manager-Radiological Health and Safety.

Section II - Purpose of Modification

The purpose of this modification is to update the Section 6.0 Administrative Controls and the organization figures of Appendix B, Section 5 to reflect recent organization and management title changes which have taken place at the Authority.

Section III- Impact of the Change

These modifications will not alter the conclusions reached in the FSAR and SER accident analysis.

Section IV - Implementation of the Modification

The modifications as proposed will not impact the ALARA or Fire Protection Program at JAF.

Section V - Conclusion

The incorporation of these modifications: a) will not increase the probability nor the consequences of an accident or malfunction of equipment important to safety as previously evaluated in the Safety Analysis Report; b) will not increase the possibility for an accident or malfunction of a different type than any evaluated previously in the Safety Analysis Report; and c) will not reduce the margin of safety as defined in the basis for any Technical Specification, and d) does not constitute an unreviewed safety question.

Section VI - References

- (a) JAF FSAR
- (b) JAF SER