

U.S. NUCLEAR REGULATORY COMMISSION
OFFICE OF INSPECTION AND ENFORCEMENT

REGION III

Reports No. 50-266/81-08; 50-301/81-07

Docket Nos. 50-266; 50-301

License No. DPR-24; DPR-27

Licensee: Wisconsin Electric Power Company
231 West Michigan Avenue
Milwaukee, WI 53201

Facility Name: Point Beach, Units 1 and 2

Inspection At: Point Beach Site, Two Creeks, WI

Inspection Conducted: March 23-27, 1981

Inspector: *[Signature]*
M. L. Gildner

4/9/81

Approved By: *[Signature]*
K. R. Baker, Chief
Management Programs Section

4/9/81

Inspection Summary

Inspection on March 23-27, 1981 (Reports No. 50-266/81-08; 50-301/81-07)

Areas Inspected: Routine, unannounced inspection of calibration and surveillance of safety-related components and equipment. The inspection involved a total of 38 inspector-hours onsite by one NRC inspector. None of the inspector-hours were spent onsite during offshifts.

Results: In the area inspected, the inspector identified one apparent item of noncompliance. The apparent item of noncompliance was a failure to follow the acceptance criteria of a major calibration procedure per Details Paragraph 4.b.

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DETAILS

1. Persons Contacted

G. A. Reed, Manager, Nuclear Power Division
*J. J. Zach, General Superintendent
*R. E. Link, Superintendent Engineering, Quality, and Regulatory
J. C. Reizenbuechler, I&C Engineer
*E. A. LeClair, I&C Supervisor
M. E. Crouch, Maintenance Supervisor

Additional plant technical and administrative personnel were contacted during the course of the inspection by the inspector.

*Denotes those personnel present during the exit meeting.

2. Test Equipment Calibration

The inspector selected the below listed test equipment used in the calibration of safety-related equipment and determined that the accuracy of those instruments was traceable to the National Bureau of Standards or other independent testing agencies; the equipment was properly controlled; and storage of equipment was adequate.

a. Test Equipment Reviewed

TI-10 Mansfield/Green Deadweight Tester Model T-50
TI-45 Heise Test Gauge Model CCM6101
TI-90 Fluke Digital Multimeter 8600A
TIS-1002 General Radio Decade Resistance 1433-F
TIS-1009 Fluke D. C. Calibrator Model 3330E

b. Findings

No items of noncompliance or deviations were identified in this subarea.

3. Surveillance of Safety Related Components or Equipment

The inspector reviewed plant surveillance/calibration procedures listed below. This review was performed to determine if the surveillance test was covered by properly approved procedures; the procedures contained prerequisites and preparations for the test; the procedures contained functional testing of instrumentation; the procedures included acceptance criteria; the procedures included operational checks prior to returning equipment to service; the technical content of the procedures would result in satisfactory testing.

a. Documents Reviewed

ICP 2.1 Reactor Coolant Flow Test
ICP 2.1 Reactor Power VS Delta T Test
ICP 2.1 Steam Generator Pressure Test
ICP 2.1 Containment Pressure Isolation Valve Test

ICP 4.1 Reactor Coolant Temperature Calibration
ICP 4.1 and ICP 4.3 Pressurizer Water Level Calibration
ICP 4.1 and ICP 4.8 Containment Spray Calibration
ICP 4.1, ICP 4.6, and ICP 4.7 Steam Generator Flow Mismatch Calibration

ICP 4.12 Accumulator Pressure Calibration
ICP 4.17 Volume Control Tank Level Calibration
ICP 4.22 Containment Sump Level Calibration
ICP 4.30 Refuel Source Range Drop-In Detector Calibration

b. Findings

No items of noncompliance or deviations were identified in this subarea.

4. Calibration of Safety Related Components Required by Technical Specifications

The inspector reviewed plant calibration records and selected logs and records for the below listed calibration activities. This review was performed to determine that the frequency of calibration was met; the service status of the system was in conformance with the applicable limiting conditions of operation; the procedures used to calibrate the components were reviewed and approved as required by the Technical Specifications; the procedures used contained acceptable trip settings using applicable technical specification requirements; the procedures used contained detailed stepwise instructions, the technical content of the procedures would result in satisfactory calibration; the trip points conformed to applicable Technical Specifications; the qualifications of two individuals in the I&C Group having responsibilities for performing calibrations were adequate.

a. Calibration Records Reviewed

Note: All records reviewed were the most recently completed test/calibration for both units.

ICP 4.2 Reactor Coolant Flow Transmitters
ICP 4.4 Pressurizer Pressure Transmitters
ICP 4.7 Feedwater Flow Transmitters
ICP 4.8 Containment Pressure Transmitters
ICP 4.11 Accumulator Level Transmitters
ICP 4.14 Boric Acid Control System
ICP 4.17 Volume Control Tank Level Transmitters
ICP 4.20 Residual Heat Removal Pump Flow
ICP 4.22 Containment Sump Level
ICP 4.29 Analog Rod Position

b. Findings

Of the twenty (20) sets of calibration data for the procedures listed above, five (5) contained as left values outside the acceptance criteria for the procedure. An additional twenty (20) sets of calibration data from "Series 4.0" procedures were examined and two (2) more similar occurrences were noted.

Noncompliance

Point Beach Nuclear Plant Technical Specification Section 15.6.8.1 states, in part that the plant shall be operated and maintained in accordance with approved procedures. Major procedures...shall be provided for the following operations where these operations involve nuclear safety of the plant. Category No. 7 is surveillance and testing of safety related equipment.

10 CFR 50 Appendix B, Section XI states in part that test results shall be documented and evaluated to assure that the test requirements have been satisfied.

Contrary to the above, the inspector found the following calibrations to be outside the test acceptance criteria with no evidence of the technician noting these conditions and the tests accepted by the I&C supervisor and the I&C engineer. Each of these tests has a section for noting problems and disposition of the problem. In each case, the area was blank. Discussions with the I&C engineer indicated that he did review results for violating technical specification limits and conservativeness.

ICP 4.7	Unit 1 dated December 4, 1980 Feedwater Flow	1FT-476 1FT-477 FI-466 FI-476
ICP 4.11	Unit 2 dated April 18, 1980 Accumulator Level	2LT-938
ICP 4.14	Unit 1 dated December 9, 1980 Boric Acid Dilution Flow	F111
ICP 4.14	Unit 2 dated May 6, 1980 Boric Acid Dilution Flow	F111
ICP 4.19	Unit 2 dated April 17, 1980 Refueling Water Storage Tank Level	LI-920
ICP 4.21	Unit 1 dated December 18, 1980 Charging Water Flow	FM-128
ICP 4.22	Unit 2 dated April 18, 1980 Containment Sump "A" Level	LT-4107

Of particular concern is ICP 4.7 for feedwater flow. Discussions on the review of this item with the I&C Engineer indicated that it was conservative from the standpoint of feed flow/steam flow mismatch, but had not considered understatement of feed flow impact on the reactor calorimetric calculations. Discussions with the reactor engineers indicated they had not been informed of the feedwater calibration errors. (266/81-08-01; 301/81-07-01)

5. Calibration of Safety Related Components Not Specified by Technical Specifications

The inspector reviewed plant calibration records listed below. This review was performed to determine if specific calibration requirements have been established; the operating range/accuracy of components were consistent with applicable Technical Specifications/SAR; the procedures used to calibrate the components were reviewed and approved as required by Technical Specifications; the procedures used contained acceptance criteria consistent with Technical Specifications/SAR criteria; the procedures used contained detailed instructions commensurate with the complexity of the calibration; the technical content of the procedures would result in satisfactory calibration.

a. Calibration Records Reviewed

ICP 5.18 Rod Speed Control
ICP 5.19 Steam Dump Control
ICP 5.20 Pressurizer Pressure Control
ICP 5.21 Pressurizer Level Control
ICP 6.12 Auxiliary Feedwater
ICP 6.15 Auxiliary Coolant
ICP 6.17 Safety Injection
PT-M-1 Station Batteries
PT-A-1 Emergency Diesel Inspection
PT-R-4 Testing of Safeguard Protective Relaying

b. Findings

Noncompliance

ICP 6.17 for Unit 1 dated November 10, 1980 left FIT-930 for Chemical Addition Tank Inlet Flow outside the test acceptance values without evidence of evaluation.

This is a further example of the problem described in Subsection 4.b.

6. Exit Meeting

The exit meeting was held with those plant personnel indicated by an asterisk in Paragraph 1 on Friday, March 27, 1981. The findings of this report were discussed with those present.

The inspector indicated to those present his appreciation of the cooperation by plant personnel during the conduct of this inspection.